



A Centenor Energy Company

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TOLEDO, OHIO 43652-0001

May 10, 1991
KB91-0285

Docket No. 50-346
License No. NPF-3

Document Control Desk
U. S. Nuclear Regulatory Commission
7920 Norfolk Avenue
Bethesda, MD 20555

Gentlemen:

Monthly Operating Report, April, 1991
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit No. 1 for the month of April, 1991.

If you have any questions, please contact Bilal Sarsour at (419) 321-7384.

Very truly yours,

A handwritten signature in cursive script, appearing to read 'Louis F. Storz'.

Louis F. Storz
Plant Manager
Davis-Besse Nuclear Power Station

BMS/tld

Enclosures

cc: Mr. Paul Byron
NRC Resident Inspector

Mr. A. Bert Davis
Regional Administrator, Region III

Mr. M. D. Lynch
NRC Senior Project Manager

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346
 UNIT Davis-Besse #1
 DATE May 10, 1991
 COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 321-7384

MONTH April, 1991

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>492</u>
2	<u>536</u>
3	<u>839</u>
4	<u>880</u>
5	<u>885</u>
6	<u>877</u>
7	<u>* 838</u>
8	<u>879</u>
9	<u>880</u>
10	<u>885</u>
11	<u>888</u>
12	<u>888</u>
13	<u>885</u>
14	<u>882</u>
15	<u>880</u>
16	<u>883</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>884</u>
18	<u>885</u>
19	<u>883</u>
20	<u>883</u>
21	<u>854</u>
22	<u>879</u>
23	<u>882</u>
24	<u>883</u>
25	<u>883</u>
26	<u>877</u>
27	<u>833</u>
28	<u>834</u>
29	<u>859</u>
30	<u>876</u>
31	<u></u>

* Based on the average of 23 hours generation due to the time change.

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

OPERATING DATA REPORT

DOCKET NO. 50-346
 DATE May 10, 1991
 COMPLETED BY Bilal Sarsour
 TELEPHONE (419) 321-7384

OPERATING STATUS

1. Unit Name: Davis-Besse #1
2. Reporting Period: April, 1991
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 925
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719.0	2,879.0	111,768
12. Number Of Hours Reactor Was Critical	719.0	2,879.0	62,000.2
13. Reactor Reserve Shutdown Hours	0.0	0.0	5,393.7
14. Hours Generator On-Line	719.0	2,879.0	59,948.3
15. Unit Reserve Shutdown Hours	0.0	0.0	1,732.5
16. Gross Thermal Energy Generated (MWH)	1,924,655	7,884,233	146,010,718
17. Gross Electrical Energy Generated (MWH)	642,619	2,637,787	48,367,804
18. Net Electrical Energy Generated (MWH)	611,281	2,509,701	45,458,329
19. Unit Service Factor	100.0	100.0	53.6
20. Unit Availability Factor	100.0	100.0	55.2
21. Unit Capacity Factor (Using MDC Net)	97.3	99.7	46.5
22. Unit Capacity Factor (Using DER Net)	93.8	96.2	44.9
23. Unit Forced Outage Rate	0.0	0.0	26.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation)

Forecast

Achieved

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-346

UNIT NAME Davis-Besse #1

DATE May 10, 1991

COMPLETED BY Bilal Sarsour

TELEPHONE (419) 321-7384

REPORT MONTH April, 1991

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1 (contd)	91-03-31	5		B	N/A	N/A	N/A	N/A	Reactor Power reduction to approximately 50% for main condenser maintenance. See Operational Summary for further details.

¹F: Forced

S: Scheduled

²Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

³Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Continuation from
Previous Month

5-Load Reduction

9-Other (Explain)

⁴Exhibit G - Instructions for Preparation of Data
Entry Sheets for Licensee Event Report (LER)
File (NUREG-0161)⁵Exhibit I - Same Source*Report challenges to Power Operated Relief Valves
(PORVs) and Pressurizer Code Safety Valves (PCSVs)

Operational Summary
April, 1991

Reactor power was maintained at approximately 60 percent power (reactor power has been reduced to 60 percent for main condenser maintenance) until 0030 hours on April 3, 1991, when reactor power was slowly increased to approximately 100 percent power, which was achieved at 0800 hours on April 3, 1991.

Reactor power was maintained at approximately 100 percent power until 2400 hours on April 20, 1991, when a manual power reduction to approximately 96 percent power was initiated to perform moderator temperature coefficient testing.

After completion of moderator temperature coefficient testing, reactor power was slowly increased to 100 percent power, which was achieved at 0400 hours on April 22, 1991.

Reactor power was maintained at approximately 100 percent power until 2015 hours on April 26, 1991, when a manual power reduction to approximately 94 percent power was initiated due to low system demands as requested by the Systems Operations Center. A further power reduction to approximately 91 percent power was initiated to perform main turbine valve testing and control rod drive exercise testing.

After completion of turbine valve testing and control rod drive exercise testing, reactor power was slowly increased to approximately 100 percent power, which was achieved at 0700 hours on April 29, 1991, and maintained at this power level for the rest of the month.

REFUELING INFORMATION

Date: April 1991

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling outage? September 1991
3. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool, and (c) the new fuel storage areas.
(a) 177 (b) 328 (c) 0
4. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increased size by: approximately 900 by 1994 is under review

5. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1996 - assuming ability to unload the entire core into the spent fuel pool is maintained