



Commonwealth Edison  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

May 10, 1991

Director of Nuclear Reactor Regulation  
United States Nuclear Regulatory Commission  
Mail Station P1-137  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for April 1991.

Very truly yours,

for G. J. Diederich  
Station Manager  
LaSalle County Station

GJD/MJC/djf

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

APRIL 1991

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION (Unit 1)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit One was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor subcritical, generator off line, L1R04 in progress.
30	2400	Reactor subcritical, generator off line, L1R04 in progress.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Technical Specification Amendment for Management Title changes and elimination of Station Control Room Engineer turnover for Shift Technical Advisor (Amendment #78).

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
91-004-00	04/01/91	Partial Group II Primary Containment isolation due to blown fuse.
91-005-00	04/10/91	Missed Technical Specification surveillance for the Primary Containment.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

C TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L00776	Hydrogen Recombiner Cooling Water Valve 1E12-F312A	Valve motor.	None.	Replaced motor.
L00931	Suppression Pool Level Switches (High) 1E22-N002A, 1E22-N002B	Inproper level switch location.	Failed to alarm on high level.	Lowered level switch.
L03779	Division 1 Post LOCA Hydrogen Monitor 1AIT-CM009	Solenoid valve.	Erratic operation.	Replaced solenoid valve.
L04008	Main Steam Turbine Differential Temper- ature Relay 1E31-K3B	Relay failed testing.	None.	Replaced relay.
L05088	Control Room Emergency Make-Up Fan Differential Pressure Indicator OPDI-VC029	Worn diaphragm.	Unable to calibrate pressure indicator.	Replaced pressure indicator.
L05245	Division III Batteries, Cell #9, 16 and 24	Low individual specific gravities.	Degraded operation.	Replaced cells.
L05380	Hydraulic Control Unit 38-11	Drain stop valve leaking.	Degraded control rod operation.	Replaced disk assembly and valve stem.
L05541	Low Power Range Monitor 16-57A	Amplifier card.	None.	Replaced amplifier card.
L05861	Reactor Water Clean Up Inboard Suction Valve 1G33-F001	Thermal overload relay.	None.	Replaced thermal overload relay.

(See attached SOR Failure Report.)  
ZCADTS/5+

C TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L05910	Residual Heat Removal Shut-Down Cooling Testable Check Equalizing Valve 1E12-F099B	Torque switch.	Valve failed to seal completely. Failed leak rate test.	Replaced torque switch.
L05962	Average Power Range Monitor, Channel D 1C51-K605GR	Averaging card cracked.	None.	Replaced averaging card.
L06392	Hydraulic Control Unit 58-31	Failure of directional control solenoid valve.	Degraded control rod operation.	Replaced solenoid valve.
L06393	Hydraulic Control Unit 38-31	Instrument block valve leaking.	Degraded control rod operation.	Replaced valve cartridge.
L06520	1B Diesel Generator Governor	Governor booster leaking.	Diesel failed to start when hot.	Replaced shutdown solenoid, sync. motor.
L06532	Inboard Main Steam Isolation Valve Drain Valve 1B21-F019	Torque switch.	Valve failed to seat completely. Failed leak rate test.	Replaced torque switch.
L06546	Hydraulic Control Unit 14-07	Instrument block valve leaking.	Degraded control rod operation.	Replaced valve cartridge.
L06551	Inboard Main Steam Line Isolation Valve 1B21-F022B	Limit switch failed to reset.	Improper indication.	Replaced limit switch.
L06591	Hydraulic Control Unit 58-35	Instrument block valve leaking.	Degraded control rod operation.	Replaced valve.



C TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L06682	Hydraulic Control Unit 42-15	Instrument block valve leaking.	Degraded control rod operation.	Replaced valve.
L06683	Hydraulic Control Unit 50-31	Instrument block valve leaking.	Degraded control rod operation.	Replaced valve cartridge.
L06844	250 Volt Charger	Ammeter and signal converter failed calibration.	Improper indication.	Replaced ammeter and signal converter.
L06952	Reactor Core Isolation Cooling Pump 1E51-C004	Pump motor brushes worn.	None.	Replaced brushes.
L80708	Low Power Range Monitor 40-41D	LPRM failed test.	None.	Replaced LPRM.
L82004	Standby Liquid Control Double Block Vent Valve 1C41-F312	Valve leaking.	None.	Replaced valve.
L92248	Main Steam Isolation Air Valve Assembly 1B21-F022D	Valve leaking.	None.	Replaced valve.
L94730	Low Power Range Monitor 48-41B	LPRM damaged.	None.	Replaced LPRM.
L95422	ADS Accum Vent Valve 11N091C	Valve leaking.	None.	Replaced valve disc.

SCR dp Switch Failure Data Sheet

Equipment Piece Number: 1E31-N009C Model Number: 102-AS-B403-NX-C1A-JJTTX6

Serial Number: 85-4-272

Application: Main Steam Line High Flow Switch

Date and Time of Discovery: 04/18/91 0500 hours

Reactor Mode: 5 (Refuel) Power Level: 0%

Calibration Tolerance: 102.0 - 104.0 PSID

Nominal Setpoint: 103.0 PSID

Action Limits: <98.2 or >107.8 PSID

Reject Limits: <94.8 or >111.2 PSID

Technical Specification

Limit: 116.0 PSID

As Found Setpoint: 109.7 PSID

Date and Time of Return to Service: 04/27/91 \_\_\_\_\_ hours

Model Number of Replacement Switch: 102-AS-B403-NX-C1A-JJTTX6

Serial Number of Replacement Switch: 86-10-558

DVR Number: 1-1-91-053

Cause: Switch setpoint was found in tolerance but the setpoint was not repeatable.

Corrective Action: The switch was replaced. The failed switch was inspected.

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE MAY 10, 1991  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD:	APRIL 1991	GROSS HOURS IN REPORTING PERIOD:	719
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):	3,323	MAX DEPEND CAPACITY (MWe-Net):	1,036
		DESIGN ELECTRICAL RATING (MWe-Net):	1,078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):	N/A		

4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	0.0	1,108.8	41,718.2
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	0.0	1,107.9	40,856.7
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWhT)	0	3,565,872	117,769,473
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	0	1,230,541	39,233,829
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	-8,183	1,171,607	37,541,689
12. REACTOR SERVICE FACTOR (%)	0.0	38.5	64.9
13. REACTOR AVAILABILITY FACTOR (%)	0.0	38.5	67.5
14. UNIT SERVICE FACTOR (%)	0.0	38.5	63.6
15. UNIT AVAILABILITY FACTOR (%)	0.0	38.5	63.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	-1.1	39.3	56.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	-1.1	37.8	54.2
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	8.3

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

05/09/91

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (HWE-Net)

DOCKET NO. 050-373  
UNIT LASALLE ONE  
DATE MAY 10, 1991  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: APRIL 1991

DAY	POWER	DAY	POWER
1	-11	17	-11
2	-11	18	-11
3	-11	19	-12
4	-11	20	-12
5	-11	21	-12
6	-11	22	-11
7	-11	23	-12
8	-11	24	-12
9	-11	25	-12
10	-11	26	-12
11	-11	27	-12
12	-11	28	-12
13	-11	29	-12
14	-11	30	-12
15	-11	31	
16	-11		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%  
(Unit 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
01	910216	S	719.0	C	1	Refueling outage LIR04.

## SUMMARY OF OPERATION:

The unit entered a scheduled refueling outage on 02/16/91.

F. UNIQUE REPORTING REQUIREMENTS (Unit 1)

1. Safety/Relief valve operations

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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(None)

2. ECCS System Outages  
(See Table 5)

3. Changes to the Off-Site Dose Calculation Manual  
(None)

4. Major changes to Radioactive Waste Treatment Systems.  
(None)

5. Indications of Failed Fuel Elements.  
(None)

(Unit 1)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
(U-0)		
127	ODG01K	Replaced cooling water relief valve.
128	ODG01K	Replaced crankcase switch and oil pressure gauge.
(U-1)		
1162	1E51-F084	Disassemble and inspect check valve.
1191	1DG034	Test relief valve.
1194	1E22-S001	Replaced relays.
1229	1E22-C003	Replaced water leg pump.
1232	1E51-F045	Inspection.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

APRIL 1991

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18



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- F. UNIQUE REPORTING REQUIREMENTS
  - 1. Safety/Relief Valve Operations
  - 2. ECCS System Outages
  - 3. Off-Site Dose Calculation Manual Changes
  - 4. Major Changes to Radioactive Waste Treatment System
  - 5. Indications of Failed Fuel Elements

## I. INTRODUCTION (Unit 2)

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit Two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761 extension 2427.

## II. MONTHLY REPORT

### A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, generator on line at 1134 MWE.
3	0100	Reduced power to 1065 MWE for weekly surveillances.
	0700	Increased power to 1133 MWE.
10	0100	Reduced power to 1075 MWE for weekly surveillances.
	1000	Increased power to 1133 MWE.
17	0000	Reduced power to 1066 MWE for weekly surveillances.
	0900	Increased power to 1134 MWE.
19	2100	Reduced power to 830 MWE for scram time testing, heater bay work and monthly surveillances.
20	0400	Reduced power to 716 MWE for rod set.
	1800	Increased power to 990 MWE.
21	0800	Reduced power to 900 MWE to remove the motor driven reactor feed pump from service and to place the turbine driven reactor feed pump in service.
22	1800	Increased power to 1130 MWE.
23	2330	Reduced power to 1060 MWE for weekly surveillances.
24	1100	Increased power to 1133 MWE.
28	0400	Reduced power to 1000 MWE for secondary containment leak rate testing.
	2100	Reduced power to 980 MWE for outboard main steam isolation valve testing.
29	1600	Increased power to 1126 MWE.
30	2330	Reduced power to 1060 MWE for weekly surveillances.
	2400	Reactor critical, generator on-line at 1060 MWE.

B. AMENDMENTS TO THE FACILITY LICENSE OR TECHNICAL SPECIFICATION

Technical Specification Amendment for Management Title changes and elimination of Station Control Room Engineer turnover for Shift Technical Advisor (Amendment #62).

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY RELATED EQUIPMENT (including SOR differential pressure switch failure reports).  
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 2)  
LER Number      Date      Description

(None.)

E. DATA TABULATIONS (Unit 2)

1. Operating Data Report.  
(See Table 2)
2. Average Daily Unit Power Level.  
(See Table 3)
3. Unit Shutdowns and Significant Power Reductions.  
(See Table 4)

C TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO  
SAFETY-RELATED EQUIPMENT

WORK REQUEST NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L06827	Intermediate range Monitor D	Preregulator bad.	None.	Replaced preregulator.
L07076	Reactor Core Isolation Broken diaphragm. Cooling Steam Flow Isolation Differential Pressure Switch 2E31-N007AA		Switch inoperable.	Replaced switch.

(See attached SOR Failure Report)

SQR dp Switch Failure Data Sheet

Equipment Piece Number: 2E31-N007AA Model Number: 103AS-B203-NX-JJTTX6

Serial Number: 86-10-3864

Application: RCIC High Steam Line Flow Isolation Switch

Date and Time of Discovery: 04/24/91 1945 hours

Reactor Mode: 1 (Run) Power Level: 100%

Calibration Tolerance: 110.4 - 112.4 "WC

Nominal Setpoint: 111.4 "WC

Action Limits: <106.0 or >116.8 "WC

Reject Limits: <102.8 or >120.0 "WC

Technical Specification

Limit: 128.0 "WC

As Found Setpoint: --- "WC

Date and Time of Return to Service: 04/26/91 1615 hours

Model Number of Replacement Switch: 103AS-B203-NX-JJTTX6

Serial Number of Replacement Switch: 90-8-6256

DVR Number: 1-2-91-016

Cause: Switch would not hold pressure during functional testing.  
During inspection the switch was found to have a hole in the diaphragm.  
The cause of the hole could not be determined.

Corrective Action: Replaced switch. Inspected failed switch.

TABLE 2  
E.1 OPERATING DATA REPORT

DOCKET NO. 050-374  
UNIT LASALLE TWO  
DATE Apr 10, 1991  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: APRIL 1991 GROSS HOURS IN REPORTING PERIOD: 719  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEMAND CAPACITY (MWe-Net): 1,036  
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

REPORTING PERIOD DATA

	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	719.0	2,079.0	39,369.2
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	719.0	2,879.0	38,726.2
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MMWh)	2,340,360	9,457,632	114,896,807
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	795,004	3,213,417	38,097,249
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	771,733	3,118,387	36,537,477
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	68.8
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	71.7
14. UNIT SERVICE FACTOR (%)	100.0	100.0	67.6
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	67.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	103.6	104.6	61.6
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	99.6	100.5	59.2
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	13.9

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
(NONE)

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3  
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 850-374  
UNIT LASALLE TWO  
DATE MAY 10, 1991  
COMPLETED BY M.J. CIALKOWSKI  
TELEPHONE (815)-357-6761

REPORT PERIOD: APRIL 1991

DAY	POWER	DAY	POWER
1	1,097	17	1,082
2	1,098	18	1,094
3	1,089	19	1,048
4	1,099	20	866
5	1,100	21	962
6	1,100	22	1,063
7	1,054	23	1,087
8	1,097	24	1,078
9	1,095	25	1,087
10	1,088	26	1,089
11	1,096	27	1,089
12	1,096	28	988
13	1,095	29	1,043
14	1,097	30	1,087
15	1,096	31	
16	1,096		



TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS >20%  
(UNIT 2)

YEARLY SEQUENTIAL DATE NUMBER	(YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
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(None)

## SUMMARY OF OPERATION:

The unit remained on line at high power throughout the month. Several minor power reductions were required for routine surveillances, heater bay work and testing.

F. UNIQUE REPORTING REQUIREMENTS (Unit 2)

1. Safety/Relief Valve Operations

<u>DATE</u> (None)	<u>VALVES</u> <u>ACTUATED</u>	<u>NO &amp; TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
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2. ECCS System Outages  
(See Table 5)

3. Changes to the Off-Site Dose Calculation Manual.  
(None)

4. Major changes to Radioactive Waste Treatment Systems.  
(None)

5. Indications of Failed Fuel Elements.  
(None)

(Unit 2)  
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
94	2E21-C002	Lubricate coupling.
96	2E12-N413B	Replace fitting on vent valve.
103	2E22-S001	Lubrication.
112	1E51-F064 1E51-F091	Administrative control.