OPERATING DATA REPORT

DOCKET NO. 50-289
DATE April 14, 1983
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: March, 1983
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.	2160.	75193.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	840.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.	0.	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.0	0.0	23840053.
19. Unit Service Factor	0.0	0.0	41.5
20. Unit Availability Factor	0.0	0.0	41.5
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	40.5
22. Unit Capacity Factor (Using DER Net)	100.0	100.0	38.7
23. Unit Forced Outage Rate			54.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup:
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-I
DATE April 14, 1983
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

MONTH March, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1983

DOCKET NO. 50-289
UNIT NAME TMI-I
DATE April 14, 1983
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	83-03-01	F	744	D	1	N/A	ZZ	ZZZZZZ	Regulatory Restraint Order

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

Operating Summary

The unit was shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. The final stages of the OTSG repair program are proceeding on schedule and plans for acceptance testing are in progress. Preparations are being made for Primary System Chemical cleaning (scheduled for May 1) and Hot Functional Testing (scheduled to start the first part of July).

MAJOR SAFETY RELATED MAINTENANCE

During the month, restart modifications continued and the following major maintenance items were performed:

The Once Through Steam Generator (OTSG) repair program continued with the flushing of both generators and the free pathing of tubes to be stabilized. The tube stabilization and plugging was completed on the "B" Generator and a drip test was performed with 22 tubes identified as leaking. Tube stabilization and plugging on the "A" Generator continued into the month of April.

The local leak rate testing program continued with the satisfactory testing of valves HM-V-1A/B, HM-V-2A/B, NI-V-26 and NI-V-27.

The pressurizer valve inspection program continued with the inspection of RC-V-1 and RC-V-17. This work included inspecting valve internals, and reassembly. The rewelding of RC-V-1 leak-off line will be completed during the month of April.

A Pressurizer vessel (RC-T-2) inspection program was commensed during the month. This included the removal of insulation, manway cover, and manway studs (for NDE inspections), and the installation of a containment tent with lights, ventilation equipment, communication equipment, etc. Preparations commensed for a camera inspection of pressurizer internals to be followed by a manned entry, Hydrolasing, follow-up camera inspection and closing of manway. This program will continue into the month of April.

Waste Gas Disposal System work continued this month with the change out of diaphragms on various WDG system valves, repairing on flange leak on WDG-V1 and repairing and resetting relief valve WDG-V-62. The WDG-V-4 replacement work continued with welding in of new valve WDG-V-158, cutting out WDG-V-4, installing new valve WDG-V-4. A helium leak test was performed satisfactorily.

Air Handling System Purge Valve seat replacement work commenced with new seats installed in AH-V-1B and AH-V-1D. Preparations for replacing the seat in valve AH-V-1A commensed and will continue into the month of April.

A snubber testing program began during the month with the following snubbers removed and tested satisfactory.

1. MS-228A
2. MS-219
3. MS-V-224

Decay heat removal pump DH-P-1A repairs (bearing and shaft replacement) commenced during the month when the pump was disassembled and inspection of pump internals was in progress as of the end of March.

REFUELING INFORMATION REQUEST

1. Name of Facility:

   Three Mile Island Nuclear Station, Unit 1

2. Scheduled date for next refueling shutdown:

   Unknown

3. Scheduled date for restart following refueling:

   Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

   If answer is yes, in general, what will these be?

   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   If no such review has taken place, when is it scheduled?

   Amendment No. 50, Cycl 3 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

   (a) 177

   (b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

   The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

   1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

OPERATING DATA REPORT

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TELEPHONE (717) 948-8551

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	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE April 14, 1983
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

MONTH March, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1983

DOCKET NO. 50-289
UNIT NAME TMI-I
DATE April 14, 1983
COMPLETED BY C. W. Smyth
TELEPHONE (717) 948-8551

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
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S: Scheduled

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Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
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4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

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   If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

   If no such review has taken place, when is it scheduled?

   Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

   (a) 177

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