

ATTACHMENT A-1

Beaver Valley Power Station, Unit No. 1  
Proposed Technical Specification Change No. 1A-190

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Revise the Technical Specifications as follows:

Remove Page

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Insert Page

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## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS

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#### 4.4.9.1

- a. The Reactor Coolant System temperature and pressure shall be determined to be within the limits at least once per 30 minutes during system heatup, cooldown, and inservice leak and hydrostatic testing operations.
- b. The Reactor Coolant System temperature and pressure conditions shall be determined to be to the right of the criticality limit line within 15 minutes prior to achieving reactor criticality.
- c. The reactor vessel material irradiation surveillance specimens shall be removed and examined in accordance with 10 CFR 50, Appendix H, to determine changes in material properties. The results of these examinations shall be used to update Figures 3.4-2 and 3.4-3.

(Proposed Wording)

ATTACHMENT A-2

Beaver Valley Power Station, Unit No. 2  
Proposed Technical Specification Change No. 2A-54

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Revise the Technical Specifications as follows:

Remove Page

3/4 4-30

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3/4 4-30

## REACTOR COOLANT SYSTEM

### 3/4.4.9 PRESSURE/TEMPERATURE LIMITS

## REACTOR COOLANT SYSTEM

### LIMITING CONDITION FOR OPERATION

3.4.9.1 The Reactor Coolant System (except the pressurizer) temperature and pressure shall be limited in accordance with the limit lines shown on Figures 3.4-2 and 3.4-3 during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:

- a. A maximum heatup of 60°F in any 1-hour period,
- b. A maximum cooldown of 100°F in any 1-hour period, and
- c. A maximum temperature change of < 5°F in any 1-hour period during hydrostatic testing operations above system design pressure.

APPLICABILITY: MODES 1, 2, 3, 4, and 5.

#### ACTION:

With any of the above limits exceeded, restore the temperature and/or pressure to within the limit within 30 minutes; perform an analysis to determine the effects of the out-of-limit condition on the fracture toughness properties of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operations or be in at least HOT STANDBY within the next 6 hours and reduce the RCS  $T_{avg}$  and pressure to less than 200°F and 500 psig, respectively, within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

#### 4.4.9.1

- a. The Reactor Coolant System temperature and pressure shall be determined to be within the limits at least once per 30 minutes during system heatup, cooldown, and inservice leak and hydrostatic testing operations.
- b. The Reactor Coolant System temperature and pressure conditions shall be determined to be to the right of the criticality limit line within 10 minutes prior to achieving reactor criticality.
- c. The reactor vessel material irradiation surveillance specimens shall be removed and examined in accordance with 10CFR 50, Appendix H, to determine changes in material properties. The results of these examinations shall be used to update Figures 3.4-2 and 3.4-3.

## ATTACHMENT B

### Beaver Valley Power Station, Unit No. 1 and 2 Proposed Technical Specification Change No. 1A-190/2A-54 REVISION OF TECHNICAL SPECIFICATION 3.4.9.1

#### A. DESCRIPTION OF AMENDMENT REQUEST

The proposed amendment would add a surveillance requirement to Technical Specification 3/4.4.9, PRESSURE/TEMPERATURE LIMITS REACTOR COOLANT SYSTEM. The new surveillance requirement would be item 4.4.9.1.c.

Surveillance Requirement 4.4.9.1.c will provide for the removal and examination of the reactor vessel material irradiation surveillance specimens in accordance with 10CFR50, Appendix H. The surveillance requirement would also state that the results of these examinations shall be used to update Figures 3.4-2 and 3.4-3.

#### B. BACKGROUND

On January 4, 1991, the NRC issued Generic Letter 91-01, Removal of the Schedule for the Withdrawal of Reactor Vessel Material Specimens from Technical Specifications. This Generic Letter provides for the removal of the schedule since it duplicates the controls on changes to this schedule that have been established by 10CFR50, Appendix H.

Prior to the issuance of Generic Letter 91-1, Beaver Valley had submitted and obtained a change to the Units 1 and 2 Technical Specification to delete both Table 4.5-5, "Reactor Vessel Material Irradiation Surveillance Schedule" and the associated Surveillance Requirement 4.4.9.1.c. This amendment request seeks to institute a reference to 10CFR50, Appendix H and reinstate the requirement that the results of the material examinations shall be used to update Figures 3.4-2 and 3.4-3 (RCS heatup and cooldown curves). This would, in effect, restore a part of the previously deleted Surveillance 4.4.9.1.c.

#### C. JUSTIFICATION

The proposed amendment request would bring Specification 3.4.9.1 into agreement with the outline of Generic Letter 91-01. It would also further ensure that Figures 3.4-2 and 3.4-3 were updated following reactor vessel sample examination. Currently the requirement for updating the heatup and cooldown curves is found only in the UFSAR.

#### D. SAFETY ANALYSIS

The proposed change is considered to be administrative in nature and will not reduce the safety of the plant. Adding the additional surveillance requirement will further ensure that Figures 3.4-2 and 3.4-3 are updated following vessel irradiated sample examination. Also, this change will not affect the reactor vessel surveillance program or alter any plant configuration or mode of operation.

#### E. NO SIGNIFICANT HAZARDS EVALUATION

The no significant hazard considerations involved with the proposed amendment have been evaluated, focusing on the three standards set forth in 10 CFR 50.92(c) as quoted below:

The Commission may make a final determination, pursuant to the procedures in paragraph 50.91, that a proposed amendment to an operating license for a facility licensed under paragraph 50.21(b) or paragraph 50.22 or for a testing facility involves no significant hazards consideration, if operation of the facility in accordance with the proposed amendment would not:

- (1) Involve a significant increase in the probability or consequences of an accident previously evaluated; or
- (2) Create the possibility of a new or different kind of accident from any accident previously evaluated; or
- (3) Involve a significant reduction in a margin of safety.

The following evaluation is provided for the no significant hazards consideration standards.

1. Does the change involve a significant increase in the probability or consequences of an accident previously evaluated?

No. The proposed change will not affect the reactor vessel material surveillance program. Implementation of the proposed change will add a surveillance requirement that will further ensure that the reactor vessel material irradiation surveillance specimens be removed and examined in accordance with 10CFR50, Appendix H. The surveillance will also further ensure that the results of the specimen examination are used to update the RCS heatup and cooldown curves of Figures 3.4-2 and 3.4-3 respectively. Therefore, these changes are considered to be administrative and do not significantly increase the probability or consequences of an accident previously evaluated.

2. Does the change create the possibility of a new or different kind of accident from any accident previously evaluated?

No. These changes will not alter any plant configuration or mode of operation. Compliance with existing regulations will ensure continued confidence in the reactor vessel material properties. Therefore, the proposed change will not create the possibility of a new or different kind of accident from any previously evaluated.

3. Does the change involve a significant reduction in a margin of safety?

No. Evaluation of the reactor vessel material for radiation embrittlement will not be altered by this change. Adding Surveillance Requirement 4.4.9.1.c would further ensure that Figures 3.4-2 and 3.4-3 were updated following reactor vessel sample examination. Therefore, the proposed amendment will not involve a significant reduction in the margin of safety.

F. NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

Based on the considerations expressed above, it is concluded that the activities associated with this license amendment request satisfies the no significant hazards consideration standards of 10 CFR 50.92(c) and, accordingly, a no significant hazards consideration finding is justified.

G. ENVIRONMENTAL EVALUATION

The proposed changes have been evaluated and it has been determined that the changes do not involve (i) a significant hazards consideration, (ii) a significant change in the types or significant increase in the amounts of any effluents that may be released offsite, or (iii) a significant increase in individual or cumulative occupational radiation exposure. Accordingly, the proposed changes meet the eligibility criterion for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22 (b), an environmental assessment of the proposed changes is not required.

ATTACHMENT C-1

Beaver Valley Power Station, Unit No. 1  
Proposed Technical Specification Change No. 1A-190

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## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS

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#### 4.4.9.1

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ATTACHMENT C-2

Beaver Valley Power Station, Unit No. 2  
Proposed Technical Specification Change No. 54

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3/4 4-30

## REACTOR COOLANT SYSTEM

### 3/4.4.9 PRESSURE/TEMPERATURE LIMITS

## REACTOR COOLANT SYSTEM

### LIMITING CONDITION FOR OPERATION

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3.4.9.1 The Reactor Coolant System (except the pressurizer) temperature and pressure shall be limited in accordance with the limit lines shown on Figures 3.4-2 and 3.4-3 during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:

- a. A maximum heatup of 60°F in any 1-hour period,
- b. A maximum cooldown of 100°F in any 1-hour period, and
- c. A maximum temperature change of  $\leq 5^\circ\text{F}$  in any 1-hour period during hydrostatic testing operations above system design pressure.

APPLICABILITY: MODES 1, 2, 3, 4, and 5.

#### ACTION:

With any of the above limits exceeded, restore the temperature and/or pressure to within the limit within 30 minutes; perform an analysis to determine the effects of the out-of-limit condition on the fracture toughness properties of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operations or be in at least HOT STANDBY within the next 6 hours and reduce the RCS  $T_{\text{avg}}$  and pressure to less than 200°F and 500 psig, respectively, within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

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