

Virginia Electric and Power Company
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117

October 5, 1994

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

NAPS: MPW
Docket No. 50-338
License No. NPF-4

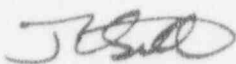
Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Unit 1.

Report No. 50-338/94-007-00

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,



J. A. Stall
Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

R. D. McWhorter
NRC Senior Resident Inspector
North Anna Power Station

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LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, DC 20503.

FACILITY NAME (1)

North Anna Power Station Unit 1

DOCKET NUMBER (2)

05000338

PAGE (3)

1 OF 3

TITLE (4)

MISSED SURVEILLANCE ON A MAIN STEAM TRIP VALVE SNUBBER DUE TO ADMINISTRATIVE PROCESS ERROR

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER	
09	14	94	94	007	00	10	04	94	FACILITY NAME	DOCKET NUMBER	
										05000	
OPERATING MODE (9)		1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
POWER LEVEL (10)		8	20.402(b)			20.405(c)			50.73(a)(2)(iv)		73.71(B)
			20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)		73.71(C)
			20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)		OTHER
			20.405(a)(1)(iii)		X	50.73(a)(2)(i)			50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
			20.405(a)(1)(iv)			50.73(a)(2)(i)			50.73(a)(2)(viii)(B)		
			20.405(a)(1)(v)			50.73(a)(2)(i)			50.73(a)(2)(x)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

Mr. J. A. Stall

TELEPHONE NUMBER (include Area Code)

(703) 894-2101

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES		NO		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
(If yes, completed EXPECTED SUBMISSION DATE)		X					

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On September 14, 1994, with Unit 1 in Mode 6 (refueling) a missed surveillance was identified associated with the visual inspection of a one and a half inch small bore hydraulic snubber on the Main Steam System. Technical Specification 4.7.10.a allows the next inspection interval to be twice the previous interval but not greater than forty eight months depending on the number of unacceptable snubbers identified during inspections. The snubber in question was believed to have been removed as a result of a design change in January 1989. The snubber database and inspection procedures were revised to reflect the snubber removal and the inspections were discontinued. This is a condition prohibited by the Technical Specifications and is reportable pursuant to 10CFR50.73 (a)(2)(i)(B).

The cause of the inadvertent deletion of the snubber from the inspection program was due to an administrative process change error which occurred in January 1989. A more thorough review of the DCP would have revealed that the snubber was to be relocated and not deleted.

No significant safety consequences resulted from this event because the snubber could perform its intended function as determined by functional testing. Therefore, the health and safety of the public were not affected at any time during the event.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

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FACILITY NAME (1)		DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
North Anna Power Station Unit 1		05000338	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
			94	007	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

1.0 Description of the Event

On September 14, 1994, with Unit 1 in Mode 6 (refueling) a missed surveillance was identified associated with the visual inspection of a one and a half inch small bore hydraulic snubber on the Main Steam System. Technical Specification 4.7.10.a allows the next inspection interval to be twice the previous interval but not greater than forty eight months depending on the number of unacceptable snubbers identified during inspections. The snubber in question was believed to have been removed as a result of a design change in January 1989. Subsequently, the snubber database and inspection procedures were revised to reflect the snubber removal and the inspections were discontinued. This is a condition prohibited by the Technical Specifications and is reportable pursuant to 10CFR50.73 (a)(2)(i)(B).

Design Change Package (DCP) 86-16 was developed to eliminate existing snubbers or to replace snubbers with rigid supports to the extent that such a change was practical. This would eliminate periodic maintenance and testing, thereby reducing man-Rem exposure, spare parts inventory, and testing costs. Modifications required by the DCP included: relocating or eliminating existing snubbers, replacing snubbers with rigid supports, and modifying existing rigid supports. The snubber in question was required to be relocated as identified in Section 3.0, Proposed Resolution, of the DCP. However, Section 3.9, In-service inspection, lists the same snubber as either being removed or replaced with a rigid restraint. During the DCP closure process procedures are revised to reflect changes that were implemented. It is believed that during this process, the snubber information was removed from the database and inspection procedures.

2.0 Significant Safety Consequences and Implications

No significant safety consequences resulted from this event because the snubber could perform its intended function as determined by functional testing. Therefore, the health and safety of the public were not affected at any time during the event.

3.0 Cause of the Event

The cause of the inadvertent deletion of the snubber from the inspection program was due to an administrative process change error which occurred in January 1989. A more thorough review of the DCP would have revealed that the snubber was to be relocated and not deleted.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
North Anna Power Station Unit 1	05000338	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
		94	007	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

4.0 Immediate Corrective Actions

The snubber was visually inspected and found to be in satisfactory condition. The snubber seals were determined to be within their 10 year seal life.

5.0 Additional Corrective Actions

Planning and Maintenance was notified and the snubber was replaced. Once removed the snubber was functionally tested satisfactorily. Revisions to the snubber database and the periodic test inspection procedures to include the snubber in future inspection intervals have been completed. In addition, a review of the snubber program was completed which verified that no other snubbers had been inadvertently omitted.

6.0 Actions to Prevent Recurrence

The current Design Change Program has controls in place that require extensive technical reviews and validations. These controls have evolved from lessons learned and were not in place at the time the snubber was inadvertently omitted. A recent review of the Design Change Process by the NRC have indicated a program that is implemented effectively.

7.0 Similar Events

None

8.0 Additional Information

Unit 2 was operating at 100 percent power (mode 1) and was not affected by this event.