AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun Station

DATE April 12, 1983

COMPLETED BY T. P. Matthews

TELEPHONE (402)536-4733

MONTH March, 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.0	17	0.0
2	0.0	18	0.0
3	0.0	19	0.0
4	0.0	20	0.0
5	0.0	21	0.0
6	0.0	22	0.0
7	0.0	23	0.0
8	0.0	24	0.0
9	0.0	25	0.0
10	0.0	26	0.0
11	0.0	27	0.0
12	0.0	28	0.0
13	0.0	29	0.0
14	0.0	30	0.0
15	0.0	31	0.0
16	0.0		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-285
DATE April 12, 1983
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

OPERATING STATUS

1. Unit Name: Fort Calhoun Station
2. Reporting Period: March, 1983
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: N/A

Notes

9. Power Level To Which Restricted, If Any (Net MWe): N/A
10. Reasons For Restrictions, If Any: None

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	2,160.0	83,401.0
12. Number Of Hours Reactor Was Critical	0.0	0.0	64,110.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	0.0	0.0	62,947.5
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	77,616,548.4
17. Gross Electrical Energy Generated (MWH)	0.0	0.0	25,735,333.5
18. Net Electrical Energy Generated (MWH)	0.0	0.0	24,330,034.4
19. Unit Service Factor	0.0	0.0	75.5
20. Unit Availability Factor	0.0	0.0	75.5
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	63.3
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	63.0
23. Unit Forced Outage Rate	0.0	0.0	3.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: April 7, 1983
26. Units In Test Status (Prior to Commercial Operation): N/A

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE April 12, 1983
COMPLETED BY T. P. Matthews
TELEPHONE (402)536-4733

REPORT MONTH March, 1983

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
82-06	821206	S	2,766	C	4	N/A	XX	XXXXXX	1982/1983 refueling outage commenced December 6, 1982.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending March 1983.

1. Scheduled date for next refueling shutdown. — March 1984

2. Scheduled date for restart following refueling. — May 1984

3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be?

      A Technical Specification Change

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — ______

   c. If no such review has taken place, when is it scheduled? — ______

4. Scheduled date(s) for submitting proposed licensing action and support information. — Methodology - Dec. 1983; Tech. Specs.- Feb. 1984

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

6. The number of fuel assemblies:

a) in the core	133	assemblies
b) in the spent fuel pool	265	"
c) spent fuel pool storage capacity	483	"
d) planned spent fuel pool storage capacity	728	"

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 1985

Prepared by J K Gasper   Date April 1, 1983

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

March, 1983
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station continued refueling outage operations during the month of March.

Reactor core loading was performed March 4 through 7 using the new loading pattern to reduce neutron fluence on the vessel. The reactor head was set March 11. Reactor coolant system heatup began March 28 reaching hot shutdown March 30.

Many system hydrostatic tests were performed including the RCS hydro at 2275 psig on March 29 and the "B" steam generator hydro on March 17.

The high pressure turbine rotor has been reassembled.

Two new personnel were added to the Operations staff March 14.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

LER Number	Deficiency
83-001	During the refueling outage the pressurizer code safety relief valves were removed and sent to an outside laboratory for a check of the setpoint pressure with steam. The setpoint for one of the two safety valves was found to be beyond the Technical Specification 2.1.6(1) limit of 2500 psia to 2545 psia ± 1%. RC-141 actually lifted at 2577 psia. The lift setpoint on the redundant valve RC-142 was within the specified tolerance.

B. CHANGES IN OPERATING METHODS

None.

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which resulted in Operation Incidents and are not reported elsewhere in the report:

Operation Incidents		Deficiency
OI-1664	ST-RPS-8, F.2	During the performance of ST-RPS-8, F.2, D/765 pressure switch found out-of-calibration.
OI-1612	ST-MSSV-1, F.1	During the performance of ST-MSSV-1, F.1, MS-275, MS-278, MS-280, MS-282 failed to actuate.
OI-1658	ST-PORV-1, F.2	During the performance of ST-PORV-1, F.2, pressure transmitter PT-115 and amplifier PM-115 found out-of-calibration.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

PROCEDURE	DESCRIPTION
SP-IC-11	Disposal of Irradiated Incore Detectors. Completed per procedure. Not an unreviewed safety question because all operations were performed in accordance with the Facility Operating License; approved written procedures; and a valid Radiation Work Permit.
SP-CSF-1	Carbon Steel Fasteners Inservice Testing. Completed per procedure. Not an unreviewed safety question because it is only an inspection.
SP-ECT-1	Eddy Current Testing of Heat Exchanger Tubes. Not an unreviewed safety question because each one of these SP's only provided for testing on non-safety related equipment.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

System Acceptance Committee Packages 1982:

Package	Description/Analysis
EEAR/FC-81-100	Main Feedwater Isolation This modification will enhance safety by providing a new safeguard signal to isolate main feedwater on certain events. Primary events which may be affected by this modification are MSLB and SB-LOCA. Main feedwater isolation reduces the severity of RCS cooldown for the MSLB, and reduces containment pressure for both events. This modification will increase plant safety and consequently, has no adverse effect on the safety analysis.
EEAR/FC-79-162	Station Lighting Transformers - 4160 V Supply This modification provides a safer and quicker means to transfer station lighting from one 4160 V bus to the other during surveillance testing and/or maintenance. This modification has no adverse effect on the safety analysis.
EEAR/FC-81-21B	Component Cooling Water Isolation to Reactor Coolant Pumps This modification ensures that component cooling water will not be isolated to the reactor coolant pumps following either a CIA signal or loss of DC power. It is acceptable to allow valves HCV-438B and HCV-438D to remain open in all cases except where a CIA signal and component cooling water pressure low signal are simultaneously present. This modification will ensure that the reactor coolant pumps will operate in a post-DBA condition while a CIA signal is present. This modification will increase plant safety and consequently, has no adverse effect on the safety analysis.
EEAR/FC-81-139	Beta Radiation Shielding (Penetration Lead Wires) This modification increases the reliability of certain safety related cables under accident and post-accident conditions. Existing cables are being modified only to provide greater radiation protection. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-81-22	Replace LCV-1173 This modification is only to replace the existing vavle with the new one. This modification has no adverse effect on the safety analysis.
EEAR/FC-83-22	Emergency Feedwater Tank Ventilation This modification does not compromise the functional ability of the tank. The integrity of the tank is being maintained by using Section VIII ASME Design Criteria and constructing the modification in accordance with that code. The probability of a failure or modification of FW-19 is not increased. This modification has no adverse effect on the safety analysis.
EEAR/FC-79-182E	Safety Injection Pump Room Level Indicators This modification installs level indicators which will provide a means to monitor excessive leakage into the sumps. This modification has no adverse effect on the safety analysis.
EEAR/FC-81-99 Part 9	Installation of Heated Junction Thermocouple Probe Holder in the Upper Guide Structure This modification installs heated junction thermocouple probe holders inside two empty CEA shrouds which protect said probe holders from normal operating cross-flow loads as well as flow-down loads. This modification has no adverse effect on the safety analysis.
EEAR/FC-81-99 Part 3	Modification of Electrical Penetrations A-10 and D-11 Modifications of these two electrical penetrations are necessary to allow for the installation of the reactor vessel level monitoring system. This modification has no adverse effect on the safety analysis.
EEAR/FC-81-99 Part 5	Installation of Modified ICI Flanges This modification was necessary in order to allow for installation of a reactor vessel level monitoring system. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-82-56	TEC Charge Converter Replacment This modification is to replace the TEC charge converter. This modification has no adverse effect on the safety analysis.
EEAR/FC-82-38	Diesel Generator 1 and 2 Vent Line Replacement This modification is to replace the cooling water vent lines in the diesel generators with new flexible hoses. These hoses will eliminate the transmission of vibrations from the diesel generators to the accessory racks. This modification has no adverse effect on the safety analysis.
EEAR/FC-82-28	Steam Generator and Pressurizer Pressure Transmitter Replacement This modification provides for replacement of the existing pressure transmitters with new nuclear qualified pressure transmitters. This modification will increase plant safety and consequently, has no adverse effect on the safety analysis.
EEAR/FC-81-99 Part 11	Machining of ICI Flanges in Place on Reactor Vessel This modification will allow for machining of the ICI flanges to restore the gasket seating surfaces thereby improving the probability of a leak-tight connection. This modification has no adverse effect on the safety analysis.
EEAR/FC-83-35	HCVS-2908 Seismic Restraint This modification modifies the support on HCVS-2908 in order to ensure that the valve will cycle properly and to ensure that it is adequately supported. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-82-143	Key-Lock Switches for Containment Purge Valve Control Circuits This modification installs new key-operated switches in panel AI-44. These switches (HC-742-1C, HC-742-1D, HC-742-2C, and HC-742-2D) are part of the containment purge valve control system. This modification will provide a better administrative control of the containment purge valves. This modification has no adverse effect on the safety analysis.
EEAR/FC-82-11	Reactor Coolant Pump Motor Cover This modification installs handles on the reactor coolant pump motor covers. This modification will have no effect on plant safety and consequently, has no effect on the safety analysis.
EEAR/FC-82-171	Bypass Valves for Check Valves SI-194, 197, 200, 203 This modification is necessary to allow for bypass valves in order to perform the safety injection hydro test. This modification has no adverse effect on the safety analysis.
EEAR/FC-78-37	Personnel Airlock Doors This modification was done in order to allow for easier personnel airlock door operation and to ensure that there is a reliable interlocking system during outages. This modification has no adverse effect on the safety analysis.
EEAR/FC-79-15	Replacement of Reactor Pressure Vessel Head Insulation This modification allows for the removal of existing RPV head insulation and the installation of new reflective type insulation. This modification has no adverse effect on the safety analysis.
EEAR/FC-82-33	Cable Spread Room Halon Dampers This modification installs halon system dampers in the cable spread room. This modification will increase safety by setting alarms and closing dampers upon manual initiation. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-82-32	Volume Control Tank Relief Repiping This modification is to replace check valve CH-284 and install a new block valve CH-534 to give added protection against radioactive gases releasing back from the volume control tank. The existing check valve CH-284 will be replaced with another better seating valve. The new block will be for isolation purposes between the volume control tank and the check valve only when the check valve needs servicing. This modification has no adverse effect on the safety analysis.
EEAR/FC-82-167	220 Volt Power for Temporary Trailers This modification was done in order to provide 220 volt power for temporary office trailers to be used during the outage. This modification is an administrative type only and has no effect on the safety analysis.
EEAR/FC-82-40	SI-222 Relief Valve Discharge This modification was done in order to reroute the discharge of SI-222 from the waste gas header to the reactor coolant drain tank. This modification has no adverse effect on the safety analysis.
EEAR/FC-80-29	Turbine Room Floor Modification This modification is for additional structural support to the grating of the turbine room floor. This modification is for the secondary plant side and has no effect on safety analysis.
EEAR/FC-81-98	Component Cooling Water Drain Valves This modification installs 3/4-inch drain valves upstream of existing valves AC-283, 284, 285 and 286 on containment coolers VA-1A, 1B, VA-8A, 8B. This modification will enhance maintenance on the component cooling relief valves thereby increasing the margin of safety. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
DCR-75B-28	AC/RW Interface Valves This modification was performed to provide air supply to the AC/RW interface valves. This modification has no adverse effect on the safety analysis.
EEAR/75B-20	Discharge Tunnel Sampling Apparatus This modification involved design of a structure to raise and lower sample nets from the discharge tunnel. This modification has no impact on the safety analysis.
EEAR/DCR-FC-76-54	Spent Fuel Pool Crosstie This modification provides for an installation of a system for a means of a crosstie between the spent fuel pool cooling system and the shutdown cooling system. This modification will have no adverse effect on the safety analysis.
EEAR/FC-79-39	Replacement of Raw Water Valves This modification provides for replacement of raw water isolation valves. This modification has no adverse effect on the safety analysis.
EEAR/FC-80-125 Part 1	Control of Gas Leaks from WD-32 This modification removed valve HG-102 and capped the line that it was in, in an effort to remove an unused part of the hydrogen gas systems. This modification has no adverse effect on the safety analysis.
EEAR/FC-82-136	Securing Containment Integrity This modification provides for installation of time delays and voltage dropping resistors on HCV-921 and 922. This modification will provide for better isolation in case of an accident. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-80-38/ 82-23/81-123	Replacment and Relocation of Containment High and Low Pressure Safety Injection Flow Transmitters This modification provides for replacement of FT-313 with a new nuclear qualified flow transmitter. This modification will increase plant safety and consequently, has no adverse effect on the safety analysis.
EEAR/FC-81-125	Reactor Coolant Pump Hatch Safety Items This modification installed hand rails and rail sockets in order to allow for easier accessibility during maintenance. This modification has no effect on plant safety or the safety analysis.
EEAR/FC-81-161 Parts 1 and 2	Safety Injection Tank Level Switches This modification provides for the installation of the safety injection tank narrow range level system. The modification of the safety injection tank instrumentation does not alter the performance of the system, but provides for a redundant system that will allow for more reliability. This modification will have no adverse effect on the safety analysis.
EEAR/FC-81-174	Access Platforms for RC-138 and RC-139 This modification installed access platforms to RC-138 and RC-139 to provide for better maintenance access. This modification has no effect on plant safety, nor on the safety analysis.
EEAR/DCR-74A-95	Hotel Waste Tank Modification This modification was done in order to discharge hotel waste tanks to overboard discharge header immediately before WD-632 so RM0-55 would monitor the waste release, but monitor tanks would be bypassed. This modification has no adverse effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-82-123	Preparation of Spare Electrical Penetration E-11 The purpose of this modification was to prepare the spare penetration E-11 so it could be used for the 1983 reactor vessel examination. This modification has no effect on plant safety, nor the safety analysis.
EEAR/FC-82-58	Battery Discharge Test Equipment The purpose of this modification was to install equipment necessary for completing battery discharge test to be completed during the refueling outage. This modification has no effect on plant safety and consequently, has no effect on the safety analysis.
EEAR/FC-82-66	Alternate Safe Shutdown/Pressurizer Heater Control This modification provides for local control of the back-up pressurizer heaters, isolation of control room circuits, and local start of back-up pressurizer heaters. This modification has no effect on plant safety and consequently, has no effect on the safety analysis.
EEAR/FC-82-57	Replacement of ASCO Solenoid Valves This modification provided for the replacement of existing ASCO solenoid valves with qualified replacements. This modification has no effect on plant safety and consequently, has no effect on the safety analysis.
EEAR/FC-83-43	Control of Heavy Loads This modification provides for redundant upper-travel limit switches on the polar crane auxiliary and main hoist. This modification has no effect on plant safety and consequently, has no effect on the safety analysis.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

Package	Description/Analysis (continued)
EEAR/FC-82-06	Load Shed of 480 Volt Raw Water Strainers This modification installed a contact start-stop switch on the raw water pump strainers. In this way, a manual restart of the raw water pump strainer will not be necessary after a load shed. This modification has no effect on plant safety and consequently, has no effect on the safety analysis.

E. RESULTS OF LEAK RATE TESTS

All leak rate tests performed during January, February and March will be reported to the PRC per a special report (which concerns itself with all CONT-2, 3, 7 and associated tests). This special report will be submitted to the PRC prior to startup.

F. CHANGES IN PLANT OPERATING STAFF

Two new operators: Harold Barnett and Tim Demanett.

G. TRAINING

Training for operators included General Employee Training, Cycle 8 Modifications, and fire brigade. Maintenance training included crane operators and signalmen, hydrostatic testing and General Employee Training.

During the 1983 Refueling approximately 400 personnel received Initial General Employee Training and approximately 450 home personnel received Annual General Employee Refresher Training.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59.

Amendment No. 69	Deletes Appendix B from the Fort Calhoun Technical Specifications on Environmental Monitoring.
Amendment No. 70	Provides Technical Specifications changes required for Cycle 8 operation.

II. MAINTENANCE (Significant Safety Related)

Refueling Outage Maintenance will be submitted as one package in April and May, 1983.

for L. T. Kusek
W. G. Gates
Manager
Fort Calhoun Station