



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

(Revised: 4/30/91)
April 23, 1991

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Attn: Document Control Desk

Subject: Byron Station Units 1 and 2
Braidwood Station Units 1 and 2
Information regarding an Application for Amendment to
Facility Operating Licenses NPF-37/66 & NPF 72/77
Appendix A, Technical Specifications
TAC# 79082/79083 and 79084/79085
NRC Docket Nos. 50-454/455 and 50-456/457

Reference: a) October 28, 1990 letter from T. Schuster to Dr. T. Murley
containing an application for Amendment to the
Byron/Braidwood Facility Operating Licenses

Dear Dr. Murley:

In Reference (a), pursuant to 10 CFR 50.90, Commonwealth Edison Company (CECo) proposed to amend Appendix A, Technical Specifications of Facility Operating Licenses NPF-37/66 and NPF-72/77 for Byron/Braidwood Stations. The proposed amendment revises a portion of Technical Specification Tables 2.2-1 and 3.3-4, Reactor Trip System Instrumentation Trip Setpoints and Engineered Safety Features Actuation System Instrumentation Trip Setpoints respectively, to reduce the setpoint for Low-Low Steam Generator Level Reactor Trip and Auxiliary Feedwater Initiation from 40.8% to 34.8% level for the Unit 1 Model D-4 Steam Generators. This setpoint reduction utilized existing excess margin present in the original 40.8% setpoint.

By this letter CECo requests that reviews and processing of the referenced application for amendment be delayed until a Setpoint Study being conducted by CECo is completed in June 1991. The scope of the study involves all setpoints provided by Westinghouse for the Byron/Braidwood Stations Technical Specifications including Steam Generator Level setpoints specified for Reactor Trip and Auxiliary Feedwater initiation. The Setpoint Study is a comprehensive review of the design assumptions made by Westinghouse comparing current plant specific data to the original Westinghouse assumptions. The study may result in changes to margin values, therefore, CECo believes the review delay to be a prudent action although the impact of the study on the Steam Generator Level setpoints is unknown at this time.

Commonwealth Edison is notifying the State of Illinois of this information relative to an application for amendment by transmitting a copy of this letter to the designated State Official.

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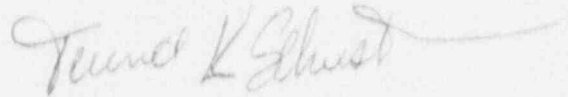
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To the best of my knowledge and belief the statements contained herein are true and correct. In some respects, these statements are not based on my personal knowledge but upon information received from other Commonwealth Edison and contractor employees. Such information has been reviewed in accordance with Company practice and I believe it to be reliable.

Please direct any questions you may have concerning this matter to this office.

Respectfully,



T.K. Schuster
Nuclear Licensing Administrator

cc: W. Kropp - Byron
S. Dupont - Braidwood
A. Hsia - NRR
R. Pulsifer - NRR
W. Shafer - RIII
Office of Nuclear Facility Safety - IDNS

State of Ill, County of Cook
Signed before me on this 23rd day
of April, 19 91 by [Signature]
Notary Public [Signature]

