



**Commonwealth Edison**

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April 8, 1983

Mr. James G. Keppler, Regional Administrator  
Directorate of Inspection and  
Enforcement - Region III  
U.S. Nuclear Regulatory Commission  
799 Roosevelt Road  
Glen Ellyn, IL 60137

Subject: Braidwood Station Units 1 and 2  
10 CFR 50.55(e) 30-day Report  
As-Built Location of NSSS  
NRC Docket Nos. 50-456/457

Dear Mr. Keppler:

On March 9, 1983, Commonwealth Edison Company Project Construction notified Mr. Harvey Wescott of your office of a potential deficiency reportable pursuant to 10 CFR 50.55(e) concerning the as-built location of the Unit 1 and 2 Steam Generators and Reactor Coolant Pump Support Columns at our Braidwood Station. For your tracking purposes, this potential deficiency was assigned number 83-03.

This letter fulfills the thirty (30) day reporting requirements of 10 CFR 50.55(e) regarding this matter and is considered a final report.

DESCRIPTION OF POTENTIAL DEFICIENCY

As a result of reviewing the as-built locations of the Unit 1 and 2 Steam Generators and Reactor Coolant Pump Support Columns (NSSS), it was determined that the locations exceeded the tolerances shown on S&L drawing S-1105.

ANALYSIS OF DEFICIENCY AND CORRECTIVE ACTION TAKEN

An analysis of the functional adequacy of the supports was made by our Architect-Engineer, S&L. Revised tolerances have since been established and the supports are currently under evaluation against these revised tolerances. The following summarizes the results of the evaluation to date:

1. The Steam Generator, Pressurizer columns, and Lateral Framing are within the revised tolerances.

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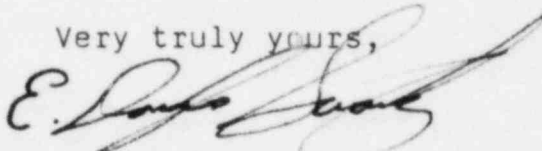
2. The Reactor Coolant Pump Lateral Framing is within the revised tolerances. The Reactor Coolant Pump columns are currently under review. Adjustment procedures have been approved for use.
3. Design drawings will be revised to incorporate the revised tolerances.

Based upon the engineering evaluation, the only rework required to establish the functional adequacy is the adjustment of the Reactor Coolant Pump columns and a check of the rotation of the knuckle.

Because this work is limited in nature and is not considered to be extensive, it is our judgement that this nonconformance is not reportable pursuant to 10 CFR 50.55(e). However, NCR No. L-489 was issued to cover this matter and the corrective rework will be performed in conjunction with other activities expected to be complete in approximately six (6) months.

Please address any questions concerning this matter to this office.

Very truly yours,



E. Douglas Swartz  
Nuclear Licensing Administrator

cc: Region III Inspector - Braidwood

Director of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

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