

# AVERAGE DAILY UNIT POWER LEVEL

Completed by L. K. Miller

Docket No. 50-272  
 Unit Name Salem # 1  
 Date April 10, 1983  
 Telephone 609-935-6000  
 Extension 4455

Month March 1983

Day Average Daily Power Level  
 (MWe-NET)

1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>

Day Average Daily Power Level  
 (MWe-NET)

16	<u>0</u>
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

P. 8,1-7 R1

8304150470 830410  
 PDR ADDCK 05000272  
 R PDR

# OPERATING DATA REPORT

Docket No. 50-272  
Date April 10, 1983  
Telephone 935-6000  
Extension 4455

Completed by L. K. Miller

## Operating Status

1. Unit Name	Salem No. 1	<u>Notes</u>
2. Reporting Period	<u>March 1983</u>	
3. Licensed Thermal Power (MWt)	<u>3338</u>	
4. Nameplate Rating (Gross MWe)	<u>1135</u>	
5. Design Electrical Rating (Net MWe)	<u>1090</u>	
6. Maximum Dependable Capacity (Gross MWe)	<u>1124</u>	
7. Maximum Dependable Capacity (Net MWe)	<u>1079</u>	
8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason _____		
N/A		

9. Power Level to Which Restricted, if any (Net MWe) None

10. Reasons for Restrictions, if any REACTOR TRIP BREAKER PROBLEMS

	<u>This Month</u>	<u>Year to Date</u>	<u>Cumulative</u>
11. Hours in Reporting Period	<u>744</u>	<u>2160</u>	<u>51169</u>
12. No. of Hrs. Reactor was Critical	<u>0</u>	<u>224.8</u>	<u>27950</u>
13. Reactor Reserve Shutdown Hrs.	<u>744</u>	<u>839.6</u>	<u>1812.7</u>
14. Hours Generator On-Line	<u>0</u>	<u>15.8</u>	<u>26663.5</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy Generated (MWH)	<u>0</u>	<u>557.9</u>	<u>79170749</u>
17. Gross Elec. Energy Generated (MWH)	<u>0</u>	<u>2760</u>	<u>25967610</u>
18. Net Elec. Energy Generated (MWH)	<u>(5339)</u>	<u>(25661)</u>	<u>24569192</u>
19. Unit Service Factor	<u>0</u>	<u>.7</u>	<u>52.1</u>
20. Unit Availability Factor	<u>0</u>	<u>.7</u>	<u>52.1</u>
21. Unit Capacity Factor (using MDC Net)	<u>0</u>	<u>0</u>	<u>44.5</u>
22. Unit Capacity Factor (using DER Net)	<u>0</u>	<u>0</u>	<u>44.1</u>
23. Unit Forced Outage Rate	<u>100</u>	<u>98.7</u>	<u>30.2</u>
24. Shutdowns scheduled over next 6 months (type, date and duration of each)			

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:  
UNKNOWN

26. Units in Test Status (Prior to Commercial Operation):

	<u>Forecast</u>	<u>Achieved</u>
Initial Criticality	<u>9/30/76</u>	<u>12/11/76</u>
Initial Electricity	<u>11/1/76</u>	<u>12/25/76</u>
Commercial Operation	<u>12/20/76</u>	<u>6/30/77</u>

8-1-7.R2

UNIT SHUTDOWN AND POWER REDUCTIONS  
REPORT MONTH March 1983

Completed by L.K. Miller

Docket No. 50-272  
Unit Name Salem No.1  
Date April 10, 1983  
Telephone 609-935-6000  
Extension 4455

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
83-030	2/25	F	838.6	A	1	---	IA	CKTBRK	Nuclear Control Rod Scram Mechanisms

1  
F: Forced  
S: Scheduled

2 Reason  
A-Equipment Failure-explain  
B-Maintenance or Test  
C-Refueling  
D-Regulatory Restriction  
E-Operator Training & Licensing Exam  
F-Administrative  
G-Operational Error-explain  
H-Other-explain

3 Method  
1-Manual  
2-Manual Scram.  
3-Automatic Scram.  
4-Continuation of  
Previous Outage  
5-Load Reduction  
9-Other

4 Exhibit G  
Instructions  
for Preparation  
of Data  
Entry Sheets  
for Licensee  
Event Report  
(LER) File  
(NUREG 0161)

5 Exhibit 1  
Salem as  
Source

## MAJOR PLANT MODIFICATIONS

REPORT MONTH MARCH 1983DOCKET NO: 50-272UNIT NAME: Salem 1DATE: April 10, 1983COMPLETED BY: L.K. MillerTELEPHONE: 609/935-6000 X4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-0723	Component Cooling	Replace shell and tube heat exchanger with titanium plate type heat exchanger to eliminate 90-10 material degradation problems.
1EC-0768	Waste Disposal Liquid	Addition of electric operators to allow remote operation of three-way valves of waste holdup and waste monitor holdup tanks and operator for cross tie (1WL115)
1EC-0776	Main Turbine Lube	Install Denison flow control valves in place of hand valves in lift oil supply lines.
1EC-0838	Chemical and Volume Control	Change pipe nipple to valve 1CV378 from Schedule 40S to Schedule 80S.
1EC-0930	Condensate Polishing	Condensate Polishing System modifications.
1EC-0975	Safety Injection	Add 100 GPM miniflow line to No. 11 and 12 Safety Injection Pumps.
1EC-1038	Auxiliary Annunciators	Add RA848A input modules to the auxiliary annunciators.
1EC-1087	Waste Disposal-Liquid	Add break flanges to the discharge lines of the #11 and 12 Containment Sump Pumps.
1EC-1211	Service Water	Provide interconnection for service water headers for the area of the chiller condensers and emergency air compressor.
1EC-1236	Service Water	Reinforce, reverse and refurbish the service water intake stop and fish gates.
1EC-1310	Control Area Air Conditioning	Install arc suppression diodes across relay and solenoid coils in control area air conditioning system.
1EC-1341	Waste Disposal (Gas)	Install two T.I. recorders on RP1.
1EC-1423	Safety Injection System	Replace narrow range transmitters.
1EC-1434	Main Steam	Replace steam traps (M-131) located downstream of 1MS902, 1MS903, and 1MS904.

\* DESIGN CHANGE REQUEST  
8-1-7.R1

## MAJOR PLANT MODIFICATIONS

REPORT MONTH MARCH 1983DOCKET NO: 50-272UNIT NAME: Salem 1DATE: April 10, 1983COMPLETED BY: L.K. MillerTELEPHONE: 609/935-6000 X4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1481	Turbine Bypass Steam (G240)	Increase the time suppression setting of the steam generator feed pump suction controls to 5.0 seconds.
1EC-1485	Electrical Penetrations	Remove following spare feed thru's from electrical penetration No. 1-57 feed thru Nos. 1 and 2-3/C #2 AWG, 17 and 18 9/C #6 AWG - use plugs and ferrules to close openings in the header plate created in place of the above feed thru's.
1EC-1504	Circulating Water	Removal of waterbox ladders on both the inlet and outlet side for the Unit 1 main surface condenser. Installation of carbon steel scaffolding supports inside inlet and outlet waterboxes.
1SC-0011	Main Turbine Lube Oil	Relocate valves for testing bearing lube oil pumps and backup seal pump.
1SC-0076	Service Water	Replace all pump room coolers with coolers of different materials.
1SC-0086	Emergency Diesel Generators	Provide a replacement heater with a better, more corrosion resistant sheath.
1SC-0104A	Main Steam Transmitters	Relocate transmitters 1PT505 and 1PT506 to a solid part of the floor.
1SC-0149	Compressed Air	Install unions to station air piping at containment penetration cooling coils.
1SC-0347	Component Cooling	Provide motor operated valves for auxiliary header isolation valves.
1SC-0464	Fire Protection	Add additional sprinkler heads in Maintenance Office storage closet.
1SC-0504	Lube Oil System	Replace 11 and 12 main turbine lube oil cooler tubing bundles with alternate materials.
1SC-0650	Steam Generator Feedwater Pumps #11 & 12	Pump journal bearings - Take new bearing retainers and modify them per Franklin Institute Research Lab Report F-A5477, page 3-4.

\* DESIGN CHANGE REQUEST  
8-1-7.R1

MAJOR PLANT MODIFICATIONS  
REPORT MONTH MARCH 1983

DOCKET NO: 50-272  
UNIT NAME: Salem 1  
DATE: April 10, 1983  
COMPLETED BY: L.K. Miller  
TELEPHONE: 609/935-6000 X4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
ISC-0749	Steam Generator Alarm Setpoint Changes	High steam generator blowdown cation conductivity alarm, change from 2.0 micromhos to 0.8 micromhos; Low steam generator blowdown pH, change from 8.4 to 8.5; Steam generator inlet dissolved oxygen alarm change from 20% of range to 5 ppb.
ISC-0760	Neutron Hi Flux at Shutdown	Temporarily disconnect the hi flux at shutdown alarm wire connected to Rack #59 in TB821-7 for duration of welding in containment during #1 Unit refueling outage.
ISC-0894	Containment Ventilation	Move test connections 1VC900 (1VC3&4), 1VC901 (1VC1&2), and 1VC902 (1VC5&6) from inside containment to north penetration area and extend same to approximately 100' in order to perform pressure drop tests as required by Tech. Spec. paragraph 4.6.3.16.
ISC-0906	Turbine Generator	Change overspeed trip setting on the Unit #1 turbine generator to 1854 rpm or 3%.
ISC-0932	Main Turbines	Install grounding braids on the shaft of #1 Main Turbine L.P.
1MD-0117	Safety Injection	Install manual isolation and drain valves for containment leak rate testing for 1SJ123, 1SJ53, 1SJ60.
1MD-0120	Demineralized Water	Install manual isolation valve.
1OD-0004	Page Installation	Install page in area of boric acid evaporator and waste evaporator control panels.
1PD-0112	Liquid Waste	Replace transmitter with a flow element and a pneumatic flow transmitter.
1PD-0140	Radiation Monitoring	Physically remove channels R6B and R20A from the Sampling Room and Chemistry Laboratory respectively.

\* DESIGN CHANGE REQUEST  
8-1-7.R1



MAJOR PLANT MODIFICATIONS  
REPORT MONTH MARCH

DOCKET NO.: 50-272  
UNIT NAME: Salem 1  
DATE: April 10, 1983  
COMPLETED BY: L.K. Miller  
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
1EC0723	The design function and operation of the component cooling system within which these heat exchangers are being replaced, has not changed from that covered in the FSAR and Technical Specifications. No unreviewed safety or environmental questions are involved.	
1EC0768	Provision of remote operations does not create any new safety hazards. The basis for the Technical Specifications are not affected. No unreviewed safety or environmental questions are involved.	
1EC0776	The installation of the expanded lift oil system and turning gear spray does not alter the original design concept of the system. No unreviewed safety or environmental questions are involved.	
1EC0838	No change to the basic design and function of the system is involved. No unreviewed safety or environmental questions are involved.	
1EC0930	These modifications are for the purpose of providing improved sampling capability. No unreviewed safety or environmental questions are involved.	
1EC0975	This change does not affect any safety related system or the safe shutdown of the reactor. No unreviewed safety or environmental questions are involved.	
1EC1038	This change does not affect safety related equipment. No unreviewed safety or environmental questions are involved.	
1EC1087	The addition of break flanges in the discharge piping does not change the Technical Specifications or the FSAR. No unreviewed safety or environmental questions are involved.	
1EC1211	This change provides the availability of more than one unit for use during an outage. No unreviewed safety or environmental questions are involved.	
1EC1236	This DCR reinforces the structural integrity of the service water stop gates. No unreviewed safety or environmental questions are involved.	

\* Design Change Request

MAJOR PLANT MODIFICATIONS  
REPORT MONTH MARCH

DOCKET NO.: 50-272  
UNIT NAME: Salem 1  
DATE: April 10, 1983  
COMPLETED BY: L.K. Miller  
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
1EC1310	This change will reduce electrical noise induced into other circuits. No unreviewed safety or environmental questions are involved.	
1EC1341	This change provides additional information to the operator and does not affect the function of the system. No unreviewed safety or environmental questions are involved.	
1EC1423	The reliability of the transmitters is being enhanced. No unreviewed safety or environmental questions are involved.	
1EC1434	Orifices have superior operational and maintainability characteristics and are a good alternative solution for the replacement of the steam traps. No unreviewed safety or environmental questions are involved.	
1EC1481	The basic function of the equipment is unchanged. No unreviewed safety or environmental questions are involved.	
1EC1485	There is no compromise in safety as a result of this change. No unreviewed safety or environmental questions are involved.	
1EC1504	Due to corrosion within the Unit No. 1 water boxes the existing ladders must be removed. It has been determined that these ladders are preventing the cathodic protection system from working properly. No unreviewed safety or environmental questions are involved.	
1SC0011	The relocation of the valves does not change the Technical Specification or the FSAR. No unreviewed safety or environmental questions are involved.	
1SC0076	Relocation to provide access for coil removal does not affect any current safety analysis nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.	
1SC0086	Replacement heaters will be less susceptible to corrosion. No unreviewed safety questions are involved.	

\* Design Change Request



MAJOR PLANT MODIFICATIONS  
REPORT MONTH MARCH

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UNIT NAME: Salem 1  
DATE: April 10, 1983  
COMPLETED BY: L.K. Miller  
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
1SC0104A	This is a safety related change because it involves moving a safety related transmitter and its associated tubing and wiring. However, no unreviewed safety or environmental questions are involved.	
1SC0149	Installation of this system does not affect any current safety analysis nor does it create any new safety hazards. No unreviewed safety questions are involved.	
1SC0347	The addition of motor operators will enable more rapid and remote operation. Handwheels will be maintained to allow manual operation. No unreviewed safety or environmental questions are involved.	
1SC0464	This change does not alter the function or operation of the fire protection system. No unreviewed safety or environmental questions are involved.	
1SC0504	The replacement tube bundles will upgrade these heat exchangers. No unreviewed safety or environmental questions are involved.	
1SC0650	This design change does not affect any presently performed safety analysis nor does it create any new safety hazards. No unreviewed questions are involved.	
1SC0749	The setpoints do not enter into the logic for the basis of any Technical Specifications. No unreviewed safety or environmental questions are involved.	
1SC-760	No unreviewed safety or environmental questions are involved.	
1SC0894	This change does not affect any presently performed safety analysis nor does it create any new safety hazards. No unreviewed safety or environmental questions are involved.	
1SC0906	This change does not affect the safe operation of the turbine generator or any safety related equipment. No unreviewed safety or environmental questions are involved.	
1SC0932	This DCR provides grounding braids for turbine shaft grounding. No unreviewed safety or environmental questions are involved.	

\* Design Change Request

MAJOR PLANT MODIFICATIONS  
REPORT MONTH MARCH

DOCKET NO.: 50-272  
UNIT NAME: Salem 1  
DATE: April 10, 1983  
COMPLETED BY: L.K. Miller  
TELEPHONE: 609/935-6000 X4455

*DCR NO.	10CFR50.59	SAFETY EVALUATION
1MD0117		Installation of the test isolation valve does not change the system operating characteristics. No unreviewed safety or environmental questions are involved.
1MD0120		The addition of these valves enhance maintenance but do not alter the basic function or design. No unreviewed safety or environmental questions are involved.
1OD0004		No safety function, analysis, or margin will be adversely affected. No unreviewed safety questions are involved.
1PD0112		This change does not affect any safety related systems or the safe shutdown of the plant. No unreviewed safety questions are involved.
1PD0140		These monitors do not perform a safety function and are not intended for use during accident conditions. No unreviewed safety or environmental questions are involved.

\* Design Change Request

SALEM UNIT NO. 1

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WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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905623	M	1	CONT. AIR LOCK, EL. 100
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FAILURE DESCRIPTION: INSPECT PER CBI RECOMMENDATIONS. ALSO REPACK SHAFTS AND REPLACE MISSING ACCESS COVER BOLTS.

CORRECTIVE ACTION: COULD NOT REPACK SHAFTS BUT TIGHTENED PACKING NUTS. REPLACED DOOR SEALS ON INNER AND OUTER DOORS.

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905644	M	1	RX FLUX THIMBLES
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FAILURE DESCRIPTION: EDDY CURRENT TEST ALL UNIT 1 FLUX THIMBLES 830111.

CORRECTIVE ACTION: TEST COMPLETE. 56 THIMBLES DONE. 2 WERE CLOGGED AND WERE NOT DONE. 830111.

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908725	M	1	VALVE, 11SW223
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FAILURE DESCRIPTION: DISASSEMBLE, INSPECT AND REPAIR AS NECESSARY.

CORRECTIVE ACTION: FOUND TUBE SHEET BADLY CORRODED DUE TO GRAPHITIZATION, TUBES AND SHROUD DAMAGED DUE TO CAVITATION. REMOVED TUBE ASSEMBLY AND INSTALLED TUBE SHEET SPACER.

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908979	M	1	VALVE, 1PR1
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FAILURE DESCRIPTION: 1PR1 LEAKS THROUGH. 821018.

CORRECTIVE ACTION: REMOVED BURRS FROM CAGE SEAT AND CLEANED VALVE INTERNALS. REPLACED CAGE AND BONNET GASKETS. 821104.

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914790	M	1	VALVE, 11VC19
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FAILURE DESCRIPTION: REMOVE VALVE 11VC19 AND RELOCATE WITH VALVE 21VC19 MARK NO. FA138. 821222.

CORRECTIVE ACTION: COMPLETED AS REQUIRED BY DR MD-82-1317. 830114.

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917279	M	1	11 CC ROOM COOLER
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FAILURE DESCRIPTION: ROOM COOLER HAS SERVICE WATER LEAK. 821004.

CORRECTIVE ACTION: OPENED COOLER AND CUT AWAY SOME PORTION OF FINNED MATERIAL TO IDENTIFY LEAK LOCATION. REPAIRED LEAK WITH BELZONA. 821120.

SALEM UNIT NO. 1

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WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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925409	M	1	VALVE, 13MS167
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FAILURE DESCRIPTION: CANNOT GET CLOSED INDICATION IN CONTROL ROOM STEAM GENERATOR BEZEL OR OVERHEAD LIGHTS.

CORRECTIVE ACTION: RESET THE RELAYS THAT WERE OPENED ON EMERGENCY SIGNAL.

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927335	M	1	FIRE BARRIER PENETRATIONS
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FAILURE DESCRIPTION: SEAL FIRE BARRIER PENETRATIONS 1419A2008S, 9S, 12S, AND 13S.

CORRECTIVE ACTION: PENETRATIONS SEALED.

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930923	M	1	PRESSURIZER CONTROL HEATERS
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FAILURE DESCRIPTION: CONTROL HEATERS KEEP TRIPPING OUT.

CORRECTIVE ACTION: CLEANED, LUBRICATED AND PERFORMED M3D ON BREAKER.

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939132	M	1	EMERG. CONTROL AIR COMP.
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FAILURE DESCRIPTION: DCP 1SC-0842 RETUBE INNER AND AFTER COOLER WITH MATERIAL AVAILABLE ON SITE. 821019.

CORRECTIVE ACTION: TUBE SHEET SENT OUT TO BE RETUBED. REINSTALLED. SEE 988097 AND 984743.

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984743	M	1	EMERG. CONTROL AIR COMP.
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FAILURE DESCRIPTION: INSPECT ALL VALVE CAGES DISCS AND ASSEMBLY PARTS FOR OXIDATION/DIRT DAMAGE. INSPECT CYL WALL WHEN CAGES REMOVED. MEASURE PISTON RING PER INST. MANUAL.

CORRECTIVE ACTION: PISTON RINGS MEASURE 1.5 WIDE HP, 2 5/16WI DE LP REPLACED GASKETS, OIL FILTER, AIR INTAKE FILTER, 2 CHECK VALVES AND CHANGED OIL. SEE WO 939132 AND 988097.

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985916	M	1	VALVE, 1GB11
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FAILURE DESCRIPTION: CONDUCT LIFT SEAL TEST.

CORRECTIVE ACTION: REPLACED VALVE WITH NEW FARRIS VALVE SET AT 152 PSIG.

SALEM UNIT NO. 1

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WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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988097	M	1	EMERG. CONTROL AIR COMP.
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FAILURE DESCRIPTION: RETUBE 1ST INNER COOLER DUE TO EDDY CURRENT RESULTS. 1 TUBE LESS THAN 20%, 55 TUBES 20-50%, 85 TUBES 50%, 8 TUBES GREATER THAN 50%.

CORRECTIVE ACTION: PULLED INNER COOLER BUNDLE, REBUILT AND REINSTALLED. SEE 939132 AND 984743.

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991788	M	1	VALVE, 1CH145
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FAILURE DESCRIPTION: VALVE LEAKS THROUGH.

CORRECTIVE ACTION: LEAK THROUGH ON 1CH149. VALVE SEATS WERE RELAPPED AND STEM WAS REPLACED ON 1CH149

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993129	M	1	.2 RCP
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FAILURE DESCRIPTION: INSPECT, CLEAN AND WATERPROOF MOTOR BEARING INSULATION AS PER WESTINGHOUSE LETTER PSE-78-559.

CORRECTIVE ACTION: CLEANED AND REINSULATED THE UPPER BRACKET DOWELS AS PER LETTER. 821109.

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993198	M	1	13 CHILLER RECIRC. PUMP
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FAILURE DESCRIPTION: MEGGER MOTOR. ARC APPEARED WHEN MOTOR WAS STARTED.

CORRECTIVE ACTION: MOTOR SHIPPED TO OUTSIDE FACILITY FOR REPAIR.

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919543	P	1	RC LOOP11 DELTA T-TAVE CHI
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FAILURE DESCRIPTION: IBS-411GH TRIP POINT OUT OF SPEC.

CORRECTIVE ACTION: REPLACED CAPACITORS C2, 3, 4, 6, 7, 8, 9, AND 10 SUMMATOR ITC-411E AND ITM-412T.

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919570	P	1	FEED FLOW CH II, 14 S/G
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FAILURE DESCRIPTION: CHANNEL OUT OF CALIBRATION. 821106.

CORRECTIVE ACTION: REPLACED SIGNAL ISOLATOR 1FM-541C. RECALIBRATED AND ADJUSTED CHANNEL. 821106.

SALEM UNIT NO. 1

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WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
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919607	P	1	VALVEOP, 14SW15
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FAILURE DESCRIPTION: DIAPHRAM LEAKING. 821214.

CORRECTIVE ACTION: REPLACED DIAPHRAM AND SET STROKE OF VALVE. 821220.

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919888	P	1	PZR LEVEL CONTROL 1LC-459D
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FAILURE DESCRIPTION: CHANNEL OUT OF CALIBRATION. 821206.

CORRECTIVE ACTION: REPLACED C2 AND C3 IN 1LC-459D. 830104.

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919946	P	1	1PRI CONTROLLER
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FAILURE DESCRIPTION: 1PC-456E1F TRIPS OUT OF SPEC. 830103.

CORRECTIVE ACTION: REPLACED COMPARATOR SNG0205 WITH SN1158. 820102.

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930331	P	1	ACCUMULATOR, #14
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FAILURE DESCRIPTION: HIGH/LOW LEVEL ALARM CH-D WILL NOT ALARM ON HIGH LEVEL.

CORRECTIVE ACTION: REPLACED C2 AND C3 IN 1LC-934D/H POWER SUPPLY BOARD. REPLACED 1LC-935D/H SW G0121 WITH SW 669.

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930514	P	1	RMS, 1R11A
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FAILURE DESCRIPTION: CHANNEL IN ALARM

CORRECTIVE ACTION: REPLACED DETECTOR AND CALIBRATED. FOUND THAT THE DETECTOR HOUSING WAS CONTAMINATED DECONNED HOUSING AND RETURNED CHANNEL TO SERVICE. 82/12/06.

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930722	P	1	FEED FLOW, 14 S/G CH2
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FAILURE DESCRIPTION: INDICATOR OUT OF SPEC.

CORRECTIVE ACTION: REPLACED CAPACITOR C2 AND C3 IN COMPARATOR 1FC541 A/B POWER SUPPLY S/N GO-128.

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985882	P	1	VALVE, 12CA330
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FAILURE DESCRIPTION: STROKE AND SET LIMIT SWITCHES AFTER CAT. PERFORMS MAINT ON VALVES.

CORRECTIVE ACTION: STROKED, CHECKED SEATING AND SET LIMIT SWITCHES



SALEM UNIT 1

OPERATIONS SUMMARY REPORT

MARCH 1983

Unit 1 was placed in cold shutdown in February following two occurrences of failure of the Reactor Trip Breakers to operate upon receiving an automatic signal from the Solid State Protection System. The unit has remained shutdown throughout March while extensive investigations of the problem have been conducted by PSE&G, the NRC, and Westinghouse. A series of meetings between PSE&G and the NRC are in progress at this time.

## REFUELING INFORMATION

COMPLETED BY: L.K. Miller DOCKET NO.: 50-272  
UNIT NAME: Salem 1  
DATE: April 10, 1983  
TELEPHONE: 609/935-6000  
EXTENSION: 4455

Month April 1983

1. Refueling information has changed from last month:  
YES \_\_\_\_\_ NO X
2. Scheduled date for next refueling: December 31, 1984
3. Scheduled date for restart following refueling: March 11, 1984
4. A) Will Technical Specification changes or other license  
amendments be required?  
YES \_\_\_\_\_ NO \_\_\_\_\_  
NOT DETERMINED TO DATE 4/1/83
- B) Has the reload fuel design been reviewed by the Station  
Operating Review Committee?  
YES \_\_\_\_\_ NO X  
If no, when is it scheduled? December 1983
5. Scheduled date(s) for submitting proposed licensing action:  
December 1983 (if required)
6. Important licensing considerations associated with refueling:  
NONE  
\_\_\_\_\_  
\_\_\_\_\_  
\_\_\_\_\_
7. Number of Fuel Assemblies:  
A) Incore 193  
B) In Spent Fuel Storage 212
8. Present licensed spent fuel storage capacity: 1170  
Future spent fuel storage capacity: 1170
9. Date of last refueling that can be discharged  
to spent fuel pool assuming the present  
licensed capacity: September 1996

8-1-7.R4