

Attachment 3

Final Test Report CRD Performance Evaluation
Testing with Driveline Misalignment

September 1994

FINAL TEST REPORT
CRD PERFORMANCE EVALUATION TESTING
WITH
DRIVELINE MISALIGNMENT

DRF B11-00604 TP&P 511.1590

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CRD PERFORMANCE EVALUATION TESTING WITH DRIVELINE MISALIGNMENT

1.0 INTRODUCTION

CRD System testing was recently completed utilizing a D-Lattice driveline configuration.

Testing was conducted to evaluate CRD scram performance under abnormal core plate and top guide misalignment conditions. The control rod and fuel bundles were also inspected for wear as a consequence of the misalignment as testing progressed.

Testing targeted BWR/4 operating conditions.

Detailed testing requirements were contained in TP&P 511.1590 Rev. A. Relevant data to be maintained in DKF B11-00604. See Appendix A for an explanation of how the actual core plate displacement configuration being used can be correlated to Hatch Unit 1.

2.0 CONCLUSION

Overall CRD scram performance was unaffected for the maximum static misalignment condition (core plate = 0.500 inch, top guide = 0.200 inch in opposite direction).

There were no significant differences in scram time with core plate misalignment of 0.500 inches with or without top guide misalignment.

Post-test inspection of the test vessel internals showed minimal wear for the conditions explored during test.

3.0 TEST CONFIGURATION

A BWR/5 CRD (S/N 7341) was used during test. The only available D-Lattice control rod (P/N 706E855G1 S/N 818) was modified previously and contained spacer pads instead of the standard pins and rollers.

The 30-inch test vessel was configured for BWR/4 operation, including a BWR/6 hydraulic control unit which was modified for BWR/4 operating parameters.

Scram testing began with the vessel internals in alignment. As testing progressed, the core plate and the top guide were misaligned per requester supplied requirements as follows:

	<u>Core Plate</u>	<u>Upper Guide</u>
Test 1	0.000	0.000
Test 2	0.250	0.000
Test 3	0.500	0.000
Test 4	0.500	0.200 (in opposite direction)

Test 4 displacements simulated the primary mode of the shroud during a seismic event.

4.0 TEST PROCEDURE AND RESULTS

4.1 CRD Functional Testing

Using the standard BWR/4 scram accumulator charge combination of 575 psig N₂ precharge, with a final water charge of 1510 psig (1510/575), a total of four accumulator scrams were accumulated. Two of the scrams were performed at zero vessel pressure and the remaining two at cold hydrostatic pressure of 1050 psig. Two full stroke drive traces, full stroke settle friction and ten collet cycles preceded the accumulator scrams.

Testing concluded with a repeat of the above tests after the vessel depressurized to ambient vessel pressure.

The above test sequence was repeated at each of the misalignment conditions. Also, for the maximum misalignment condition, two accumulator scrams were conducted at 850 psig cold hydrostatic vessel pressure.

CRD performance was monitored as testing progressed. Testing under BWR/4 operating conditions was completed satisfactorily. CRD performance data ranges are summarized on the next page.

Test Result Summary

Data Range						
Misalignment (inch)	Vp	Scram (sec)	Drive (in/sec)		Settle Fr (psig)	
Core Plate	Top Guide	Psig	90%	Up	Down	Notch 00-48
0.000	0.000	20	1.860-1.950	2.97-3.17	2.86-2.92	43-48
0.000	0.000	1050	2.252-2.282			
0.250	0.000	20	1.910-1.966	3.17-3.21	2.86-2.90	37-48
0.250	0.000	1050	2.228-2.242			
0.500	0.000	20	1.918-1.962	3.10-3.23	2.88-2.91	35-45
0.500	0.000	1050	2.263-2.284			
0.500	0.200	20	1.906-1.928	3.10-3.18	2.84-2.91	35-47
0.500	0.200	850	2.430-2.445			
0.500	0.200	1050	2.246-2.258			

The above results showed that with a core plate misalignment of 0.500 inches with and without top guide misalignment, the scram data indicated no significant difference.

At the conclusion of testing additional data points were taken for core plate misalignment of 0.625 inches while holding the top guide at 0.200 inches. The additional data points are summarized below:

Core Plate	Top Guide	Psig	90%	Up	Down	Notch 00-48
0.625	0.200	20	1.870-1.932	2.99-3.08	2.73-2.80	27-45
0.625	0.200	850	2.500-2.600			
0.625	0.200	1050	2.305-2.326			
0.625	0.200	700*	2.933-3.163			
0.625	0.200	850*	2.758-2.791			
0.625	0.200	1050*	2.586-2.624			18-51

* Hot Testing

Complete results of the operating data are shown in Appendix B of this report.

4.2 Visual Inspection Of Internals

Following completion of each test phase, the vessel internals (control rod, fuel bundles) were removed and visually inspected for evidence of test damage.

Visual inspection of the control rod as testing progressed showed no obvious damage. No major dings or nicks were noted. The fuel channels showed normal wear. Observed scratches had no obvious depth.

Sketch of core plate and top guide misalignment is shown in Appendix C. Also shown are sketches and photographs of observed scratches on the control rod and fuel channels.

5.0 RECOMMENDATION

None, data submittal only.

APPENDIX A

TEST CORRELATION TO HATCH UNIT 1

The CRD performance evaluation testing with driveline misalignment data can be used in the same manner as the previous allowable static core plate displacement.

From GENE-771-44-0894, the allowable dynamic displacement (DD) of the Hatch Unit 1 core plate is equal to:

$$DD = 0.9 \times 2.5 SD / SF_{\min}$$

$$SD = \text{allowable static displacement from test} \geq 0.5''$$

$$SF_{\min} = \text{minimum safety factor}$$

$$DD = 1.125 / SF_{\min}$$

$$DD_{\text{upset}} \geq 0.5''$$

$$DD_{\text{emergency}} \geq 0.75''$$

$$DD_{\text{fault}} \geq 1.0''$$

APPENDIX B

CRD OPERATING DATA

TABLE										SCRAM DATA						
TITLE <u>Core Deflection</u>										TEST REQUEST <u>511-1590 Rev A</u>						
DRIVE MODEL <u>BWR 15</u> S/N <u>7341</u> ROD WT. (lb) _____																
COOLING PRESS (PSIG) _____ TEST SYSTEM <u>30"</u> GPM AT _____ PSIG _____										SCRAM COUNTER _____						
CUMULATIVE SCRAM NUMBER	VESSEL PRESSURE (PSIG)	TEST STEP NO.	ACCUMULATOR PRESSURE (PSIG)	PROBE (°F)	TEMPERATURE	START OF MOTION SCRAM SIG. LOSS TO DO-48 (SEC)	DO-48 TO DO-45	TRUE 5% STROKE START OF MOTION SCRAM SIG. LOSS TO DO-48 (SEC)	10% STROKE SCRAM SIG. LOSS TO DO-43 (SEC)	20% STROKE SCRAM SIG. LOSS TO PU 38 (SEC)	50% STROKE SCRAM SIG. LOSS TO DO-24 (SEC)	90% STROKE SCRAM SIG. LOSS TO DO-05 (SEC)	BUFFER TIME (SEC) DO-03 TO DO-05	Peak DO Pressure Peak PU Pressure	VESSEL WATER CONDUCTIVITY (MEG OHM)	REMARKS
1	20	6.15	575	67	.160	.120	.250	.350	.520	1.090	1.870	.370	435	75		Baseline 1 of 2
2	20	6.15	575	67	.150	.120	.240	.340	.510	1.060	1.860	.370	420	65		
3	1050	6.16	575	68	.156	.166	.281	.422	.641	1.727	2.282	.620	1000	0		Baseline
4	1050	6.16	575	68	.150	.162	.272	.410	.621	1.703	2.252	.632	1000	0		
5	20	6.17	575	68	.159	.122	.251	.354	.528	1.101	1.950	.207	405	50		
6	20	6.17	575	68	.156	.122	.248	.348	.520	1.086	1.926	.209	415	50		
7	20	6.26	575	68	.161	.118	.250	.351	.526	1.099	1.951	.208	390	50		0.25"
8	20	6.26	575	68	.161	.119	.250	.341	.530	1.110	1.966	.206	415	50		Core Plate
9	1050	6.27	575	69	.155	.160	.275	.412	.627	1.705	2.242	.612	1000	0		
10	1050	6.27	575	69	.153	.159	.272	.405	.620	1.293	2.228	.605	1000	0		
11	20	6.28	575	69	.153	.105	.233	.350	.520	1.082	1.910	.319	1000	0		
12	20	6.28	575	69	.164	.120	.243	.350	.520	1.081	1.915	.315	450	50		
13	20	6.36	575	71	.164	.120	.254	.357	.532	1.105	1.962	.325	440	0		0.500"
14	20	6.36	575	71	.158	.119	.247	.349	.523	1.091	1.938	.320	440	0		Core Plate
15	1050	6.37	575	71	.155	.164	.278	.417	.632	1.713	2.284	.338	1050	0		
16	1050	6.37	575	71	.152	.161	.273	.409	.625	1.704	2.263	.330	1050	0		
17	20	6.38	575	71	.158	.120	.248	.349	.523	1.091	1.936	.320	400	75		0.500
18	20	6.38	575	71	.155	.119	.244	.345	.515	1.078	1.918	.318	435	45		Core Plate
19	20	6.46	575	71	.166	.117	.254	.353	.524	1.087	1.922	.316	435	60		0.500
20	20	6.46	575	71	.159	.116	.246	.347	.518	1.081	1.915	.318	435	55		Core Plate
21	850	6.47	575	71	.150	.178	.281	.440	.672	1.420	2.445	.670	875	40		0.200
22	850	6.47	575	71	.152	.175	.283	.435	.678	1.405	2.430	.670	835	45		Upper Guide
23	850	6.47	575	71	.158	.160	.278	.413	.622	1.300	2.258	.615	1050	0		
24	850	6.47	575	71	.152	.157	.270	.403	.610	1.289	2.246	.612	1050	0		
25	20	6.48	575	71	.164	.117	.252	.353	.524	1.088	1.928	.318	425	64		
26	20	6.48	575	71	.161	.116	.248	.347	.517	1.076	1.906	.314	440	15		
27	20	6.56	575	70	.164	.117	.252	.351	.523	1.091	1.933	.319	435	60		
28	20	6.56	575	70	.159	.118	.248	.348	.519	1.088	1.932	.319	420	60		0.625
29	20	6.57	575	74	.158	.180	.293	.440	.680	1.440	2.500	.680	865	0		Core Plate
30	20	6.57	575	74	.150	.180	.285	.440	.675	1.435	2.600	.685	870	0		0.200
31	20	6.57	575	74	.152	.161	.273	.409	.624	1.327	2.705	.681	1140	0		Upper Guide
32	20	6.57	575	74	.158	.162	.280	.417	.634	1.338	2.324	.627	1135	0		
33	20	6.58	575	72	.155	.115	.241	.340	.510	1.075	1.908	.315	425	50		
34	20	6.58	575	72	.160	.110	.243	.340	.510	1.055	1.870	.310	460	80		
35	700	6.58.1	575	176	.156	.170	.284	.429	.682	1.692	3.163	.751	750	80		Recorder
36	700	6.58.1	575	159	.124	.184	.262	.407	.651	1.581	2.933	.616	750	110		E.W. Henderson
37	850	6.58.1	575	183	.155	.192	.299	.459	.711	1.599	2.758	.617	835	110		
38	850	6.58.1	575	168	.153	.189	.295	.451	.714	1.601	2.791	.632	870	110		
39	1050	6.58.1	575	180	.159	.176	.291	.433	.666	1.727	2.629	.554	1050	50		
40	1050	6.58.1	575	179	.157	.179	.291	.435	.667	1.768	2.586	.541	1015	55		

TEST TITLE Core Plate Deflection
 TEST SYSTEM EWS 4 TEST REQUEST NO 511.1590 CRD SERIAL NO 7341

DATE	8/25/94	8/25/94	8/24/94	8/31/94	8/30/94	8/30/94	8/31/94	8/31/94	9/1/94	9/1/94	9/2/94			
TIME	09:50	12:10	08:50	17:20	14:40	09:30	09:35	14:25	14:30	09:30	11:00			
T.R. STEP	6.13	6.17	6.24	6.28	6.38	6.34	6.44	6.48	6.58	6.54	6.58.2			
PROBE TEMPERATURE (°F)	68	68	69	69	71	70	71	71	73	70	549			
VESSEL PRESSURE	20	20	20	20	20	20	20	20	20	20	1050			
TEST OPERATOR	U.B.	V.B.	V.B.	B.F.	RAY Loui	RAY Loui	V.B.	U.B.	V.B.	V.B.	U.B.			
DATA RECORDER	B.F.	E.W.H.	F.W.H.	B.F.	B.F.	B.F.	B.F.	E.W.H.	U.B.	B.F.	U.B.			
NOTCH														
48	45	45	48	37	35	35	42	42	41	42	28.7			
46	45	45	48	44	45	45	46	47	45	45	50			
44	44	44	47	44	45	45	45	45	45	44	49			
42	44	44	47	43	45	45	45	45	45	43	50			
40	43	44	48	44	45	45	45	45	45	43	51			
38	43	44	47	44	44	44	45	45	45	43	51			
36	44	44	47	44	44	43	45	45	40	40	49			
34	43	43	46	45	43	43	44	44	35	35	35			
32	43	44	47	45	43	43	43	43	32	27	26			
30	43	44	46	45	42	42	37	38	31	29	18			
28	43	45	46	45	40	40	36	37	27	28	18			
26	43	44	46	46	38	40	35	35	27	28	25			
24	43	43	45	45	38	38	35	36	30	30	22			
22	44	44	45	44	38	38	36	37	30	30	24			
20	43	44	45	44	38	37	36	36	31	29	23			
18	44	44	45	44	38	37	37	37	31	30	23			
16	44	44	45	44	37	38	37	38	33	33	25			
14	44	44	46	44	38	36	37	38	33	33	28			
12	45	45	45	45	38	36	37	39	35	33	30			
10	45	45	45	45	38	37	37	39	36	34	38			
08	45	45	45	45	39	38	38	39	37	35	26			
06	45	46	45	44	39	38	38	40	37	35	30			
04	45	47	45	43	39	37	37	40	37	38	33			
02	45	47	45	43	38	38	39	41	37	38	35			
00	45	45	45	45	43	42	43	44	41	39	40			

CRD S/N

SETTLE FRICTION DATA

TABLE 1

COLLET RESEAT PRESSURE (-PSID)

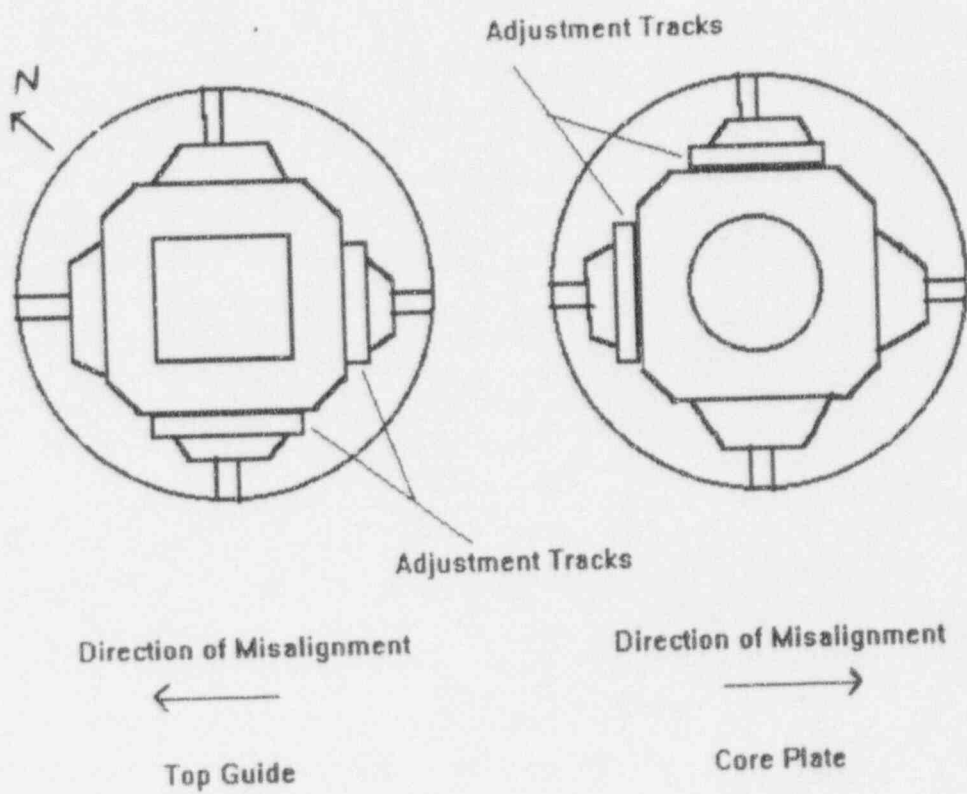
TP511-

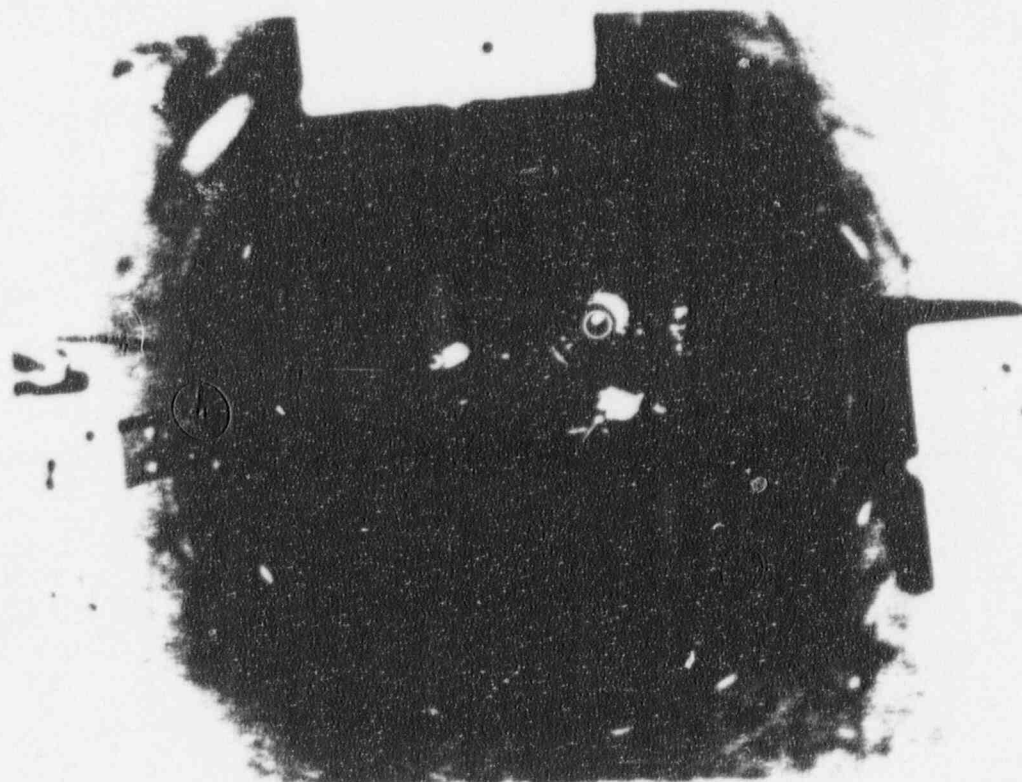
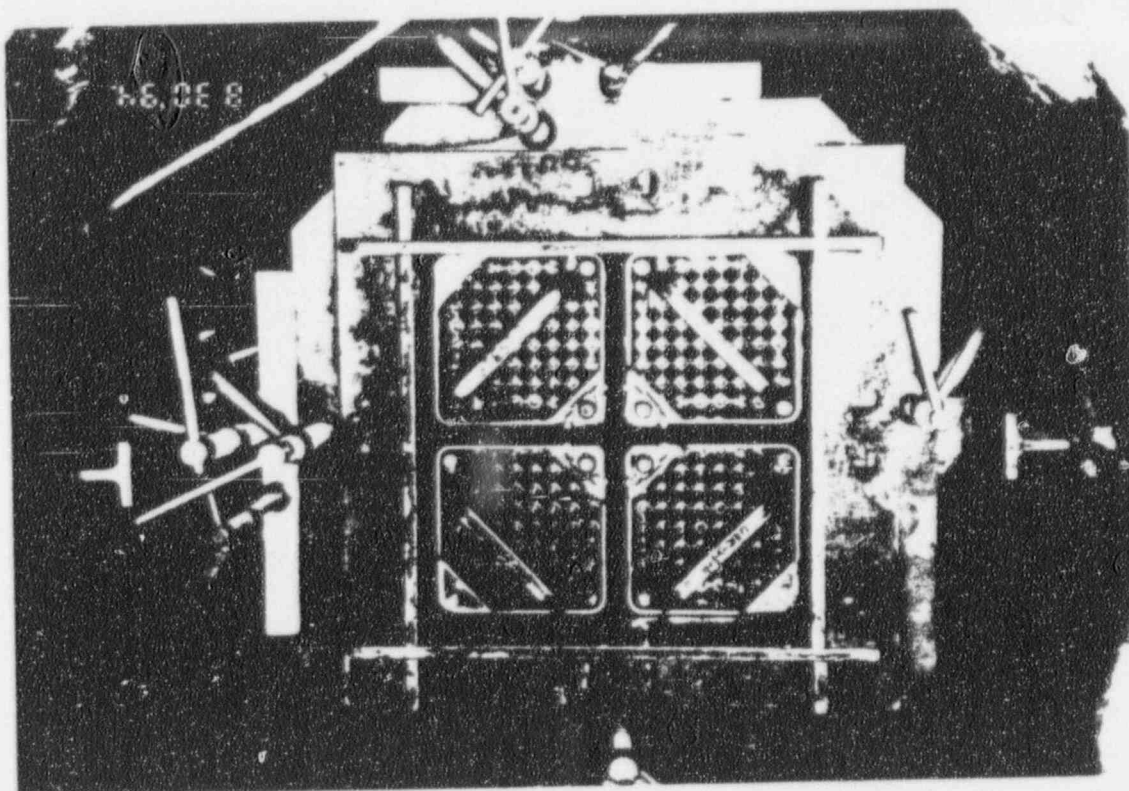
1590 Rev "A"

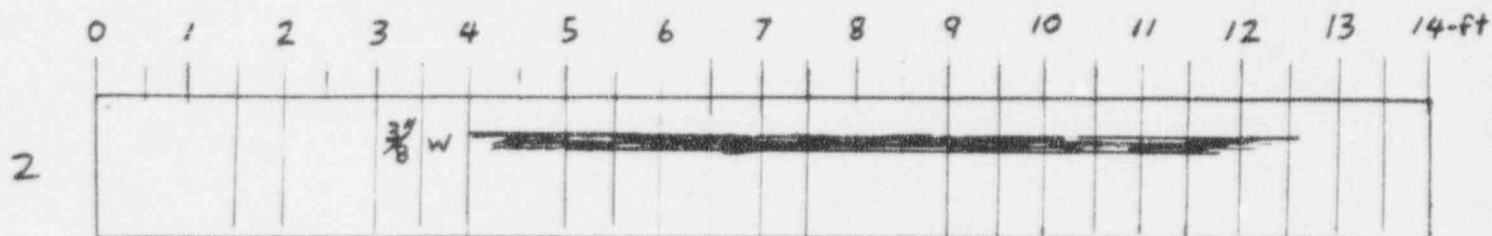
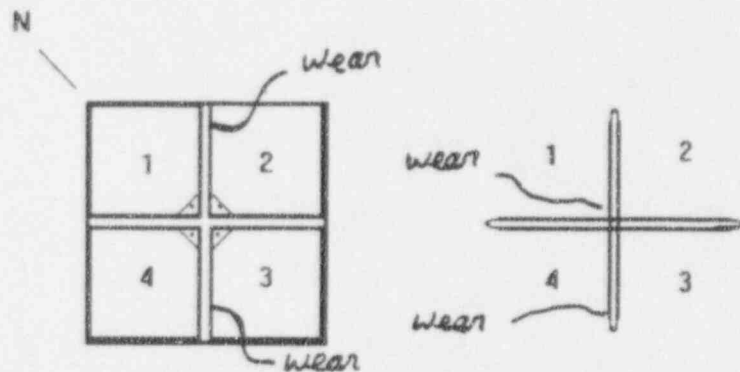
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APPENDIX C

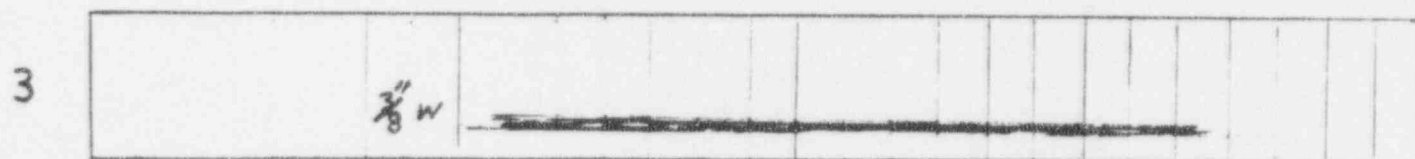
MISALIGNMENT AND WEAR RESULTS



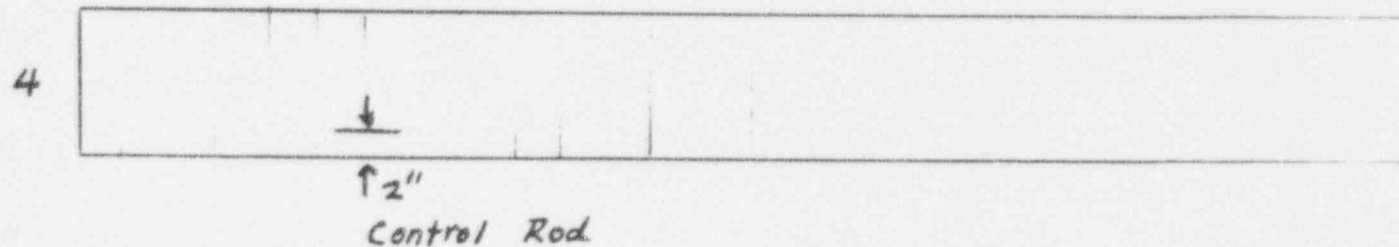
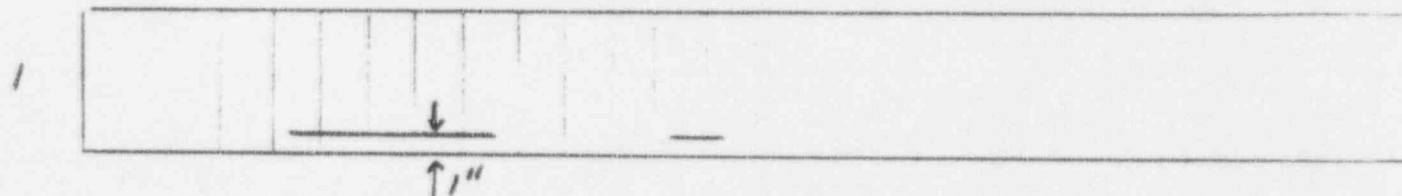




TOP



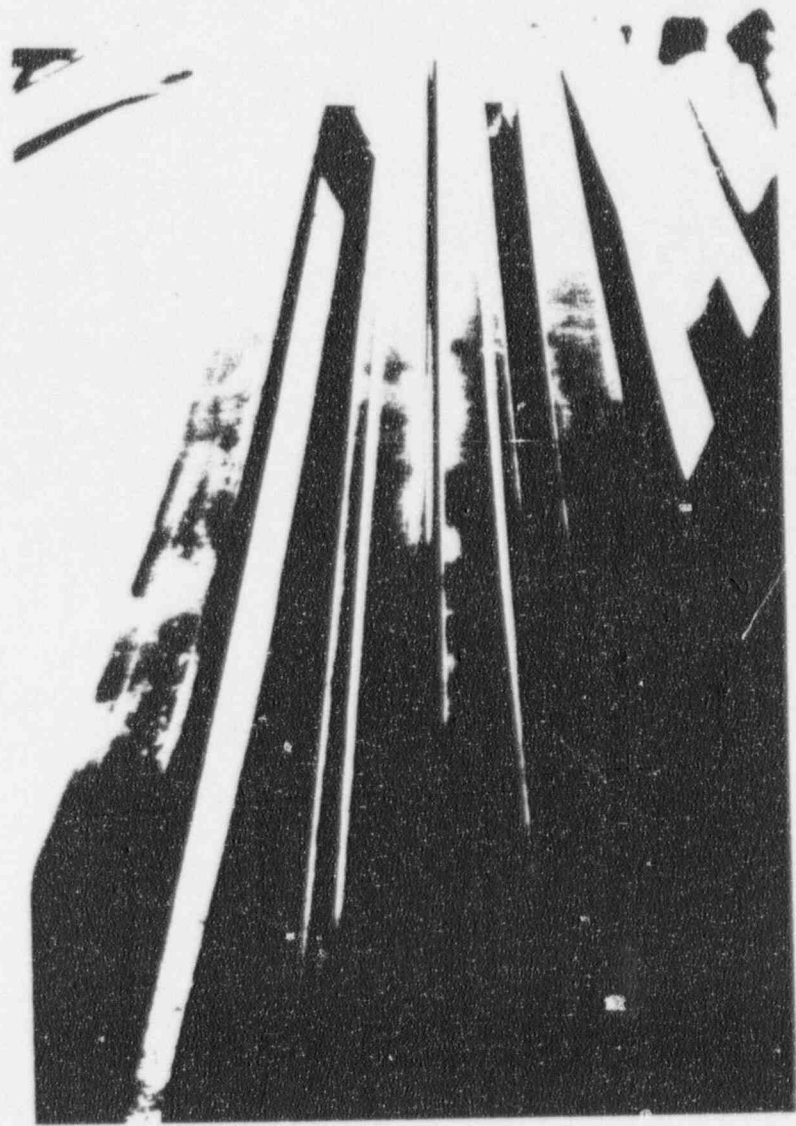
80 mil Fuel Channels



Control Rod

Wear After .5 + .2 Misalignment





Attachment 4

Hitachi Testing Report Summary

HITACHI TESTING REPORT SUMMARY

Testing was performed in the mid 1970's by Hitachi to determine the scram characteristics of a control rod during fuel channel deflections, which simulated seismic conditions. The testing is documented in HIGE-701-T8. This is a summary of that test report. The test facility is shown in Figures 3-a, 3-b, and 4. The actuator (Figure 3-b) was used to dynamically excite the four fuel assemblies, control rod, control rod drive and supports. The horizontal displacement of the fuel assemblies was measured during the excitation. The control rod was scrammed while the fuel was vibrating. The time versus control rod position was measured. Excitation both parallel to and at 45 degrees from the control rod principal axes was considered.

The control rod drive that was used in this test was a BWR/6 drive. A BWR/6 drive can be used to represent a BWR/4 by reducing the hydraulic control unit pressure. Test case II represents such a reduced pressure. Figures 15, 17, 19, and 21 give the test results for test case II. Figure 15 shows scram time versus % insertion for various fuel mid span (single sided) dynamic displacements in a direction parallel to control rod wing. Figure 19 shows the same information for displacement at a 45° angle to the control rod wings. Figure 17 shows the same information as Figure 15, except it is plotted differently. Only case II applies. Similarly Figure 21 is a replot of Figure 19. These results are judged to represent how a BWR/4 control rod would scram during a seismic event.

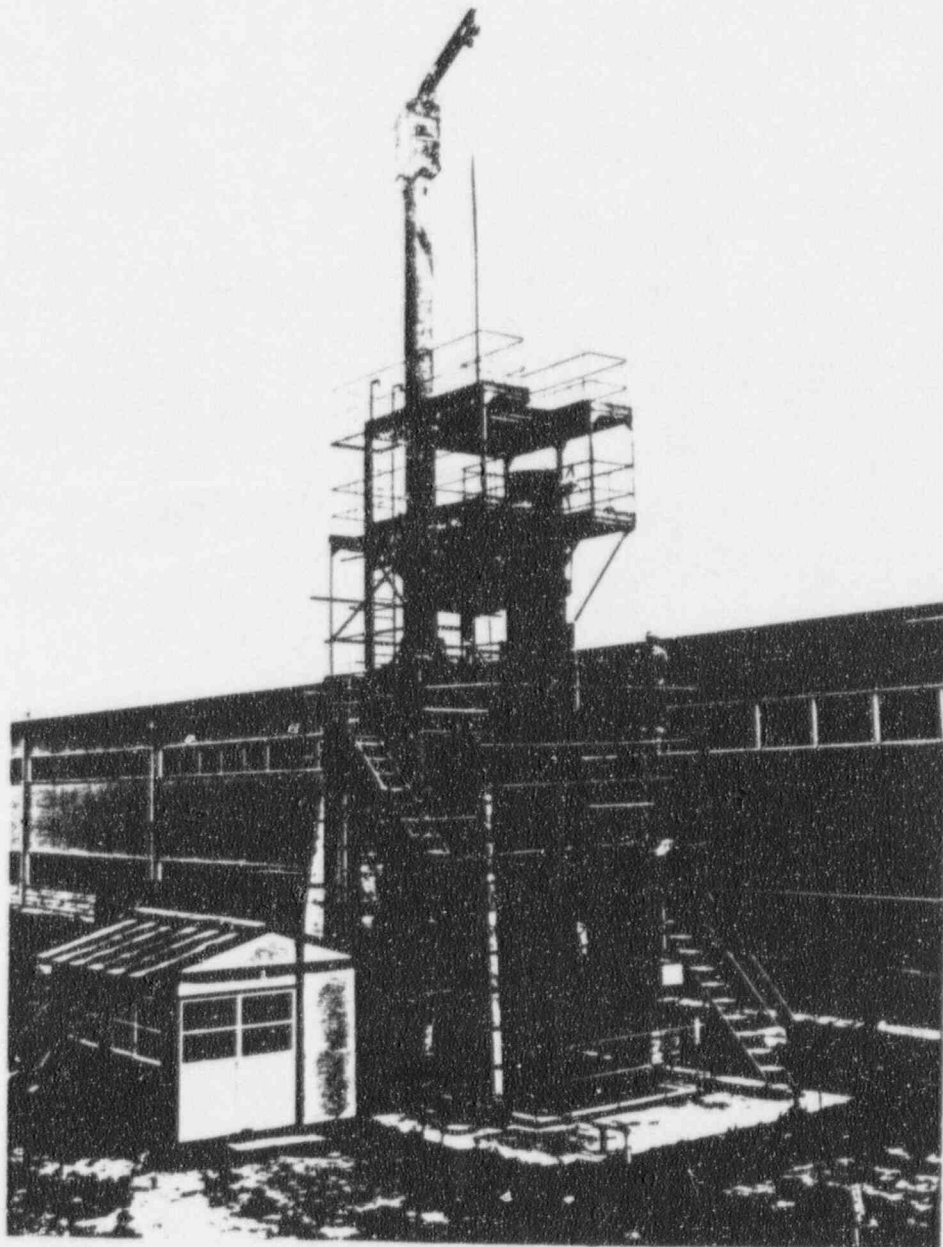


Fig. 3-a Whole View of Test Fixture

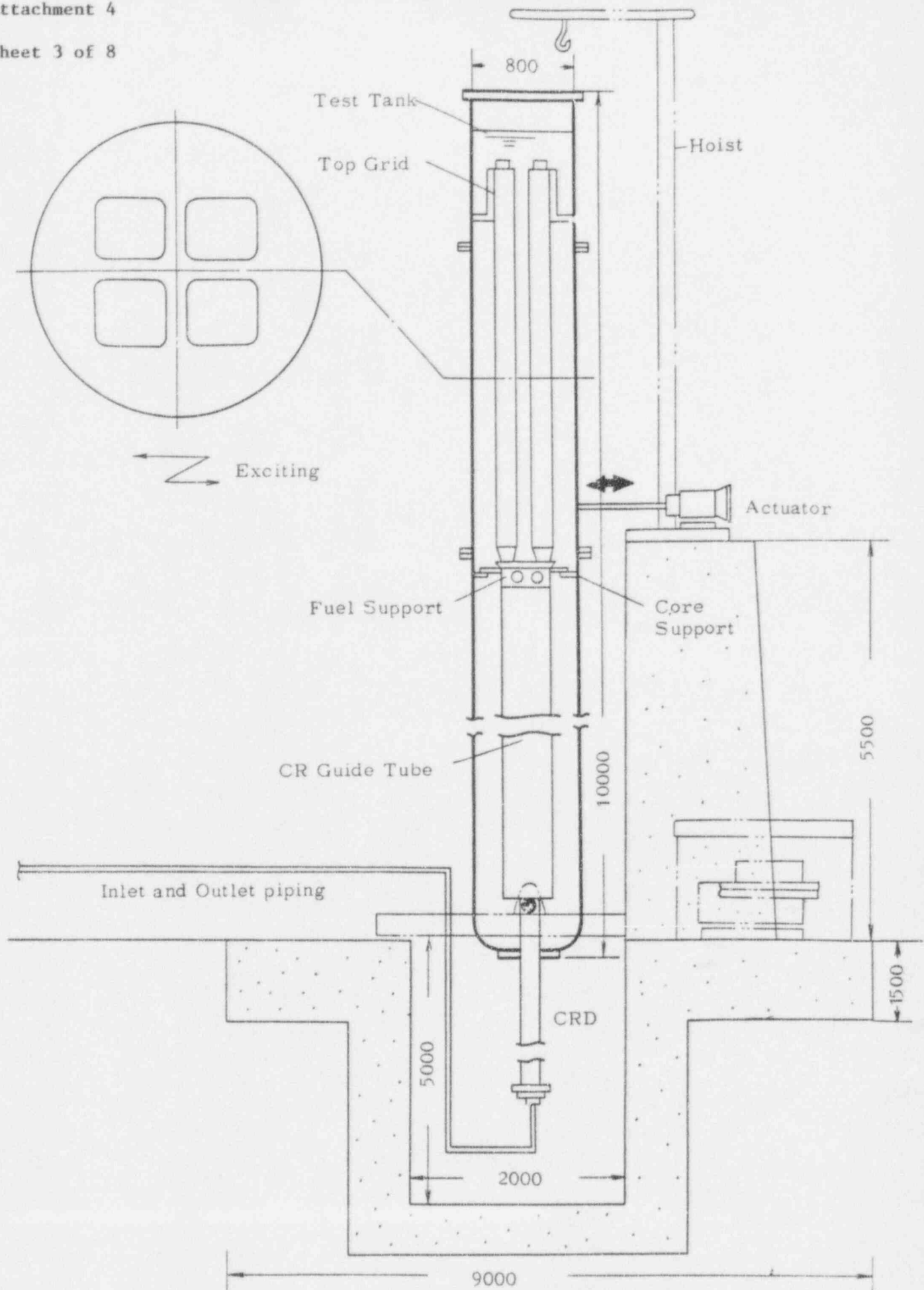
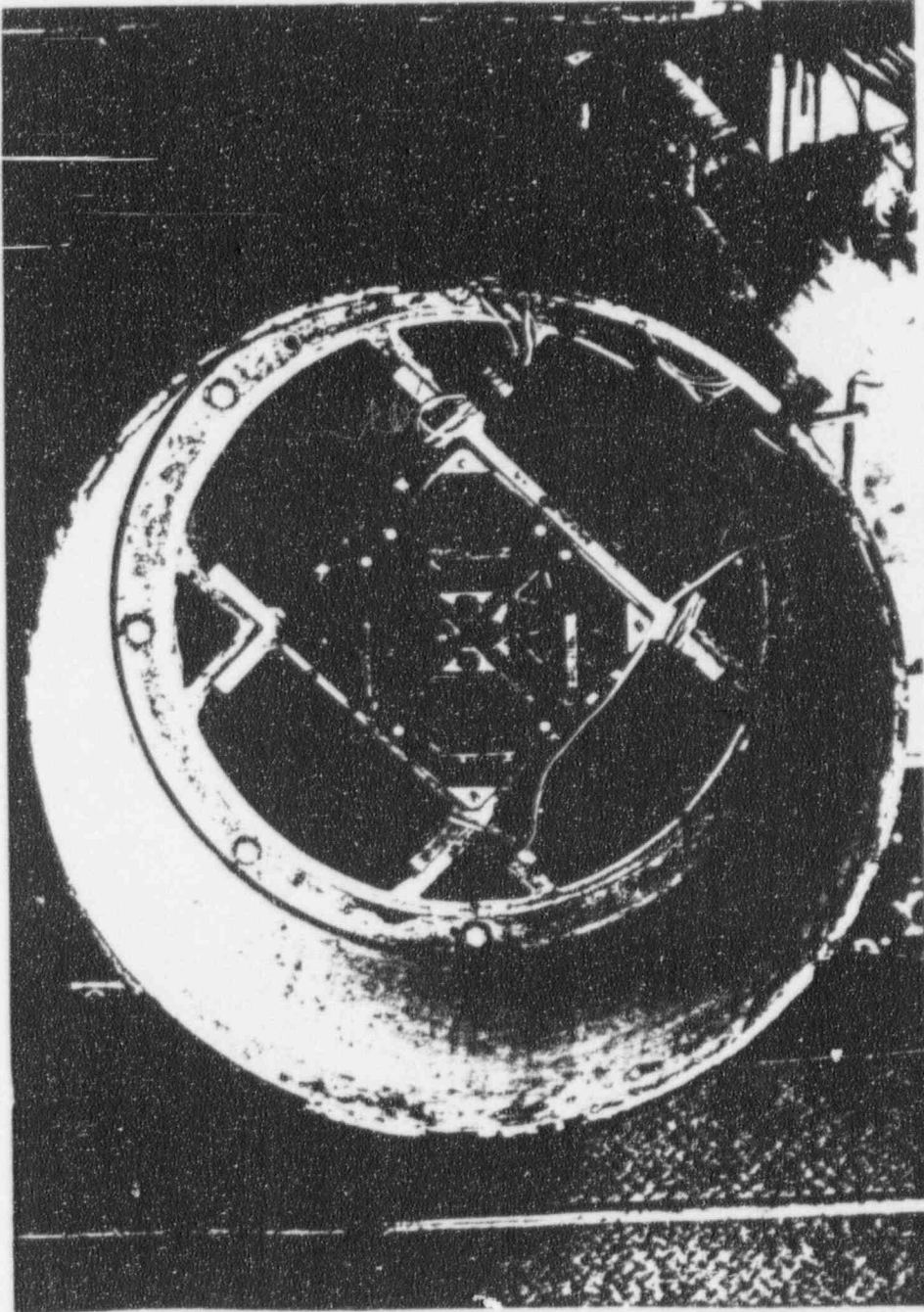


Fig. 3-b Scrammability Test Fixture

Fig. 4 Fuel Assemblies and Top Grid



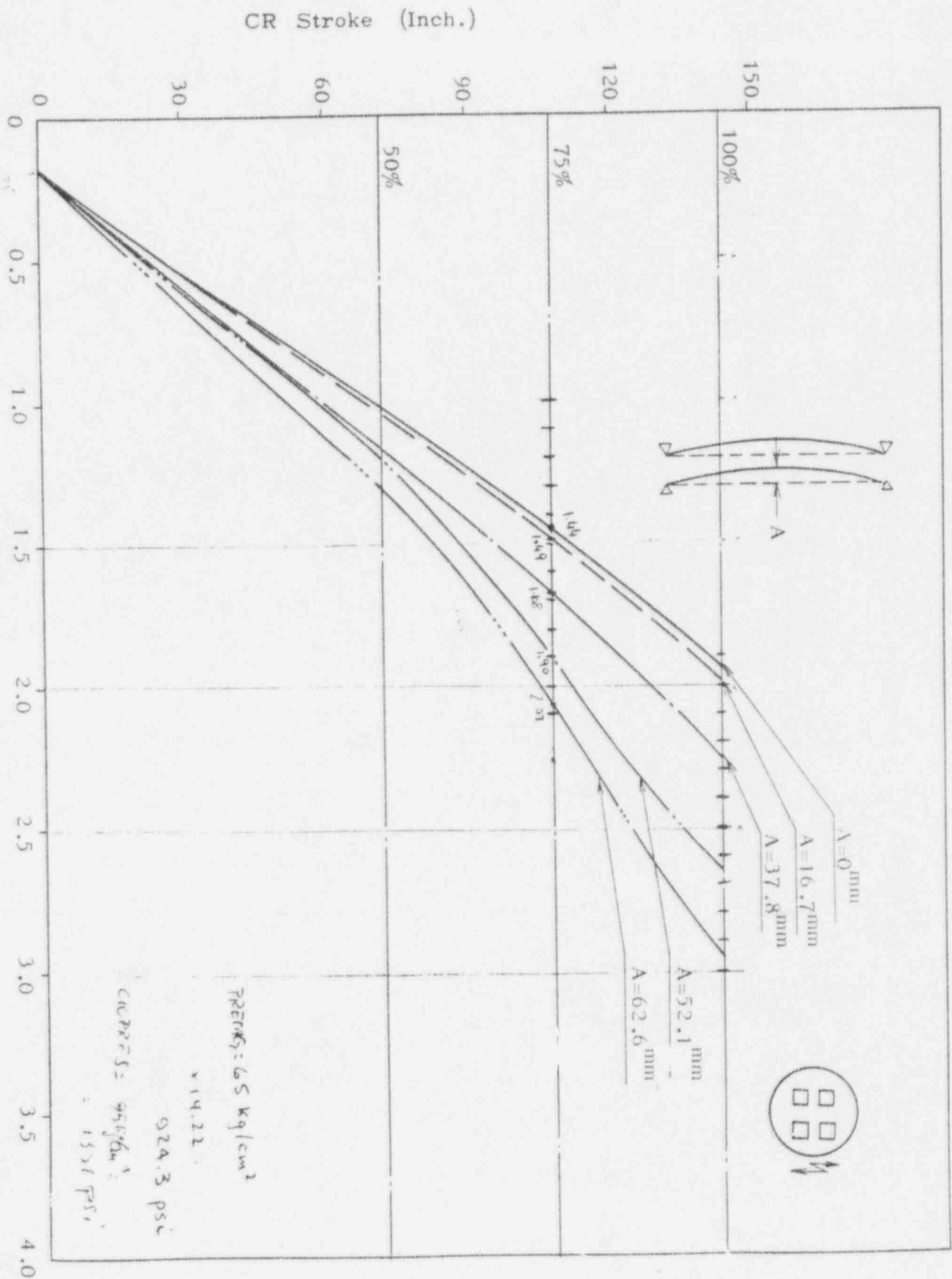


Fig. 15 Scram Locus (Case II)

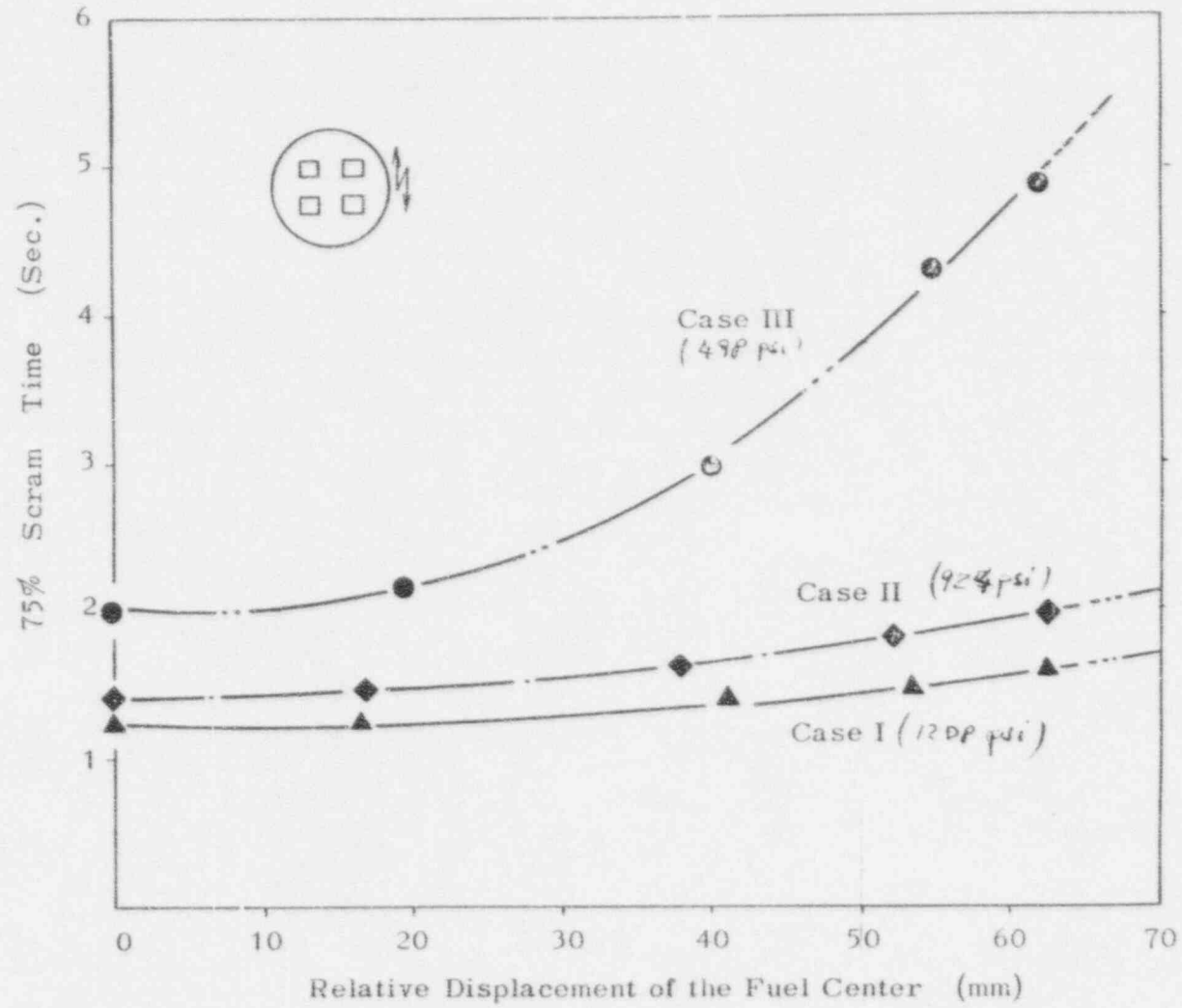


Fig. 17 Scram Time Required 75% Insertion (Parallel Excitation)

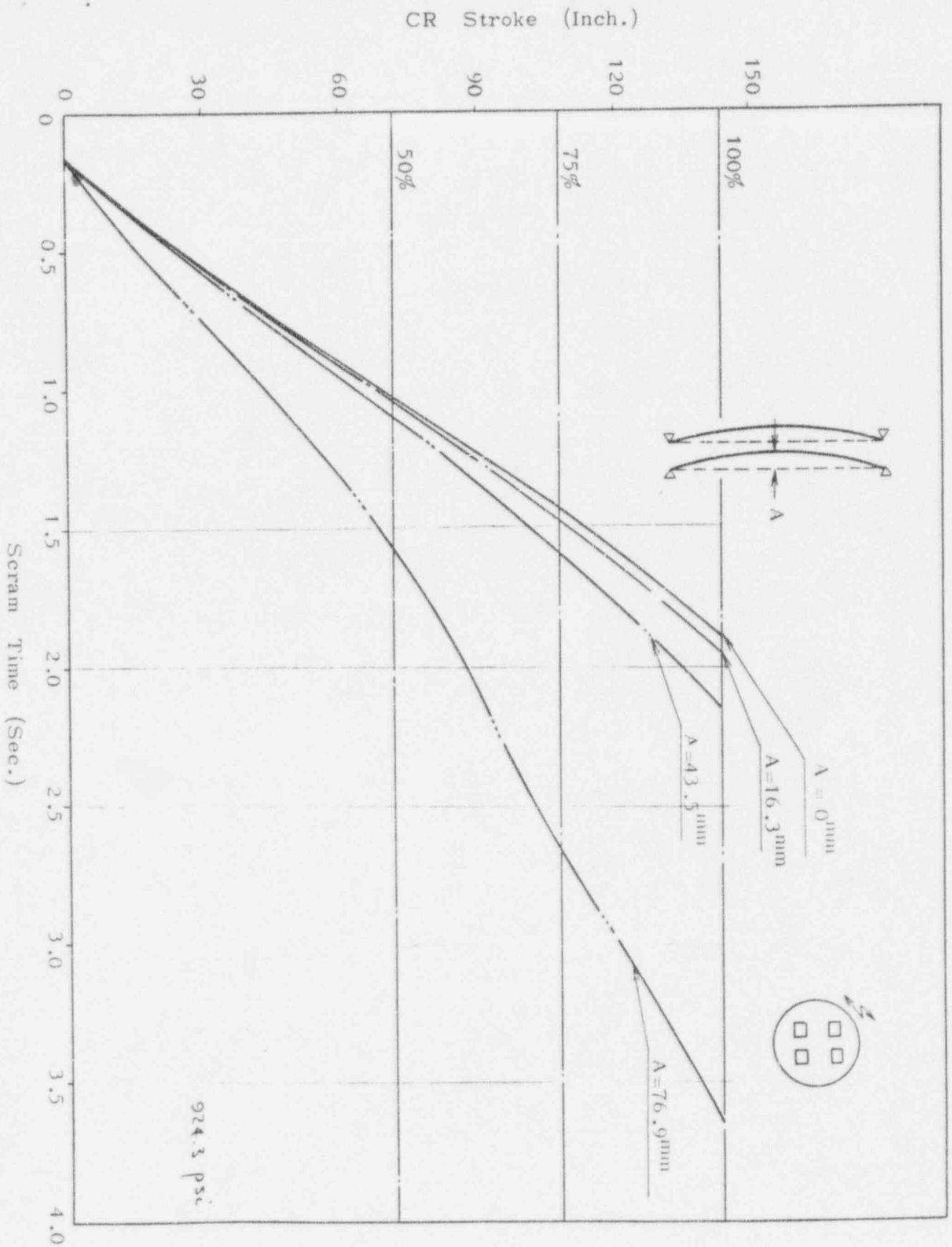


Fig. 19 Scram Locus (Case II)

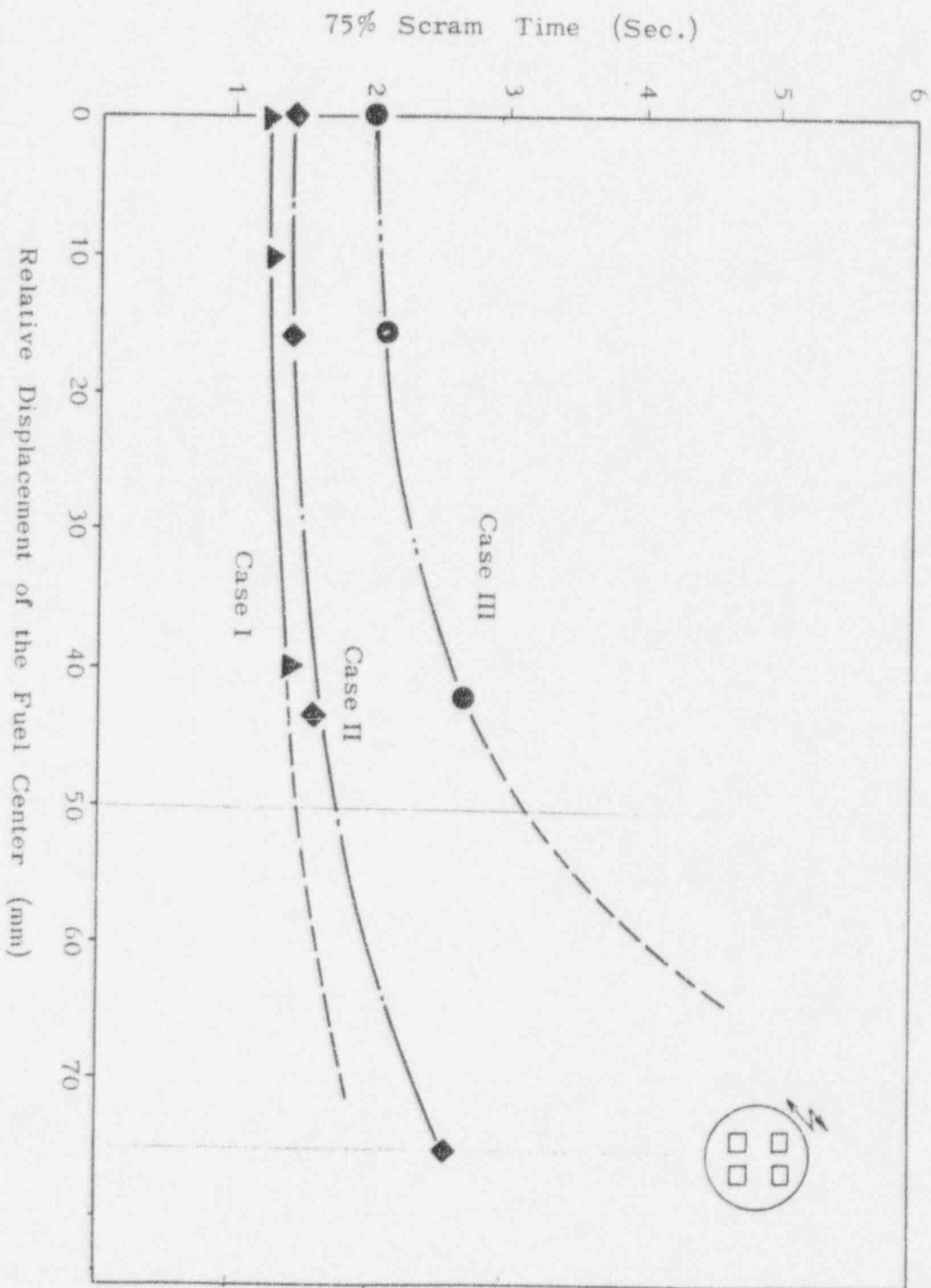


Fig. 21 Scram Time Required 75% Insertion (Diagonal Excitation)

Attachment 5

Final Stress Intensities and Displacement

Attachment 5

Hatch Unit 1 Final Stress Intensities and Displacements Sheet 1 of 4

Classification	Load Combinations	Component	Pm	Pm allow	Pm + Pb	Pm + Pb allow
Upset	OBE + P + T + W	Lower Spring	35,500	Sm = 47,500	67,500	1.5Sm = 71,250
Emergency	DBE + P + W	Lower Spring	32,100	1.5Sm = 71,250	90,900	2.25Sm = 106875
Emergency	1/2 SME + P + W	Lower Spring	39,600	1.5Sm = 71,250	99,400	2.25Sm = 106875
Faulted	1/2 SME + MSL LOCA + W	Lower Spring	39,600	2.0Sm = 95000	99,400	3Sm = 142,500
Upset	OBE + P + T + W	Upper Bracket	14,300	Sm = 47,500	35,200	1.5Sm = 71,250
Emergency	DBE + P + W	Upper Bracket	18,300	1.5Sm = 71,250	44,900	2.25Sm = 106875
Emergency	1/2 SME + P + W	Upper Bracket	29,200	1.5Sm = 71,250	71,800	2.25Sm = 106875
Faulted	1/2 SME + MSL LOCA + W	Upper Bracket	33,500	2Sm = 95,000	94400	3Sm = 142,500
Upset	OBE + P + T + W	Tie Rod	14,600	Sm = 22,800	14,600	1.5Sm = 34,200
Emergency	DBE + P + W	Tie Rod	18,600	1.5Sm = 34,200	18,600	2.25Sm = 51,300
Emergency	1/2 SME + P + W	Tie Rod	26,400	1.5Sm = 34,200	26,400	2.25Sm = 51,300

Attachment 5

Hatch Unit 1

Final Stress Intensities and Displacements

Sheet 2 of 4

Classification	Load Combinations	Component	Pm	Pm allow	Pm + Pb	Pm + Pb allow
Faulted	1/2 SME + MSL LOCA + W	Tie Rod	36,600	2Sm = 45,600	36,600	3Sm = 68,400
Emergency	LOCA ONLY	Tie Rod	24,000	1.5Sm = 34,200	24,000	2.25Sm = 51,300
Upset	OBE + P + T + W	Upper Spring	19,900	Sm = 47,500	47,600	1.5Sm = 71,250
Emergency	DBE + P + W	Upper Spring	39,600	1.5Sm = 71,250	94,700	2.25Sm = 106,875
Emergency	1/2 SME + P + W	Upper Spring	45,300	1.5Sm = 71,250	106,300	2.25Sm = 106,875
Faulted	1/2 SME + MSL LOCA + W	Upper Spring	45,300	2.0Sm = 95,000	106,300	3Sm = 142,500
Upset	OBE + P + T + W	Shroud	4,000	Sm = 16,900	10,800	1.5Sm = 25,350
Emergency	DBE + P + W	Shroud	9,200	1.5Sm = 25,350	21,300	2.25Sm = 38,000
Emergency	1/2 SME + P + W	Shroud	7,800	1.5Sm = 25,350	20,000	2.25Sm = 38,000
Faulted	1/2 SME + MSL LOCA + W	Shroud	18,700	2Sm = 33,800	46,200	3Sm = 50,700

Attachment 5

Hatch Unit 1

Final Stress Intensities and Displacements

Sheet 3 of 4

Classification	Load Combination	Component	Transient Radial Calculated	Displacements Allowable	Permanent Radial Calculated	Displacements Allowable
Upset 2	OBE + P + T + W	Lower Spring	0.27	0.75	0	0.33
Emergency 1	DBE + P + W	Lower Spring	0.56	1.12	0	0.5
Emergency 2	W + MSL LOCA	Lower Spring	0	1.12	0	0.5
Emergency 3	1/2 SME + P + W	Lower Spring	0.95	1.12	0.4	0.5
Emergency 4	W + RL LOCA	Lower Spring	0	1.12	0	0.5
Faulted 1	1/2 SME + MSL LOCA + W	Lower Spring	0.95	1.49	0.4	0.66
Faulted 2	1/2 SME + RL LOCA + W	Lower Spring	0.52	1.49	0	0.66

Attachment 5

Hatch Unit 1

Final Stress Intensities and Displacements

Sheet 4 of 4

Classification	Load Combination	Component	Transient Radial Calculated	Displacements Allowable	Permanent Radial Calculated	Displacements Allowable
Upset 2	OBE + P + T + W	Upper Spring	0.27	N/A	0	0.93
Emergency 1	DBE + P + W	Upper Spring	1.5	N/A	0	1.4
Emergency 2	W + MSL LOCA	Upper Spring	0	N/A	0	1.4
Emergency 3	1/2 SME + P + W	Upper Spring	2.0	N/A	0.2	1.4
Emergency 4	W + RL LOCA	Upper Spring	0	N/A	0	1.4
Faulted 1	1/2 SME + MSL LOCA + W	Upper Spring	2.0	N/A	0.2	1.86
Faulted 2	1/2 SME + RL LOCA + W	Upper Spring	2.0	N/A	0.2	1.86

Attachment 2

GENE-771-44-0894 Rev. 1
Justification of Allowable Displacements
of the Core Plate and Top Guide
Shroud Repair



GE Nuclear Energy

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Manager
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September 7, 1994

Mr. Bruce McLeod
Georgia Power Co.

SUBJECT: GE Nuclear Energy Report Number GENE-771-44-0894 Rev 1, "Justification for Allowable Displacements of the Core Plate and Top Guide Shroud Repair"
Dated 9/2/94

Dear Mr. McLeod:

For your information and use GE is submitting the subject document. This incorporates and addresses the comments from the VIP Repair Committee.

It should be noted that one comment that is not addressed is the one relative to the basis of the safety factors. These factors were set by the FSAR's and are beyond the scope of this report.

This report is being provided under the Terms and Conditions of the agreement between the Electric Power Research Institute and the General Electric Co., version EPRI/GE93 930120.

Please free to call the writer at (408) 925 1842 should you have any questions.

Very Truly yours,


C.W. Dillmann

cc. Robert Thomas EPRI
J.E. Charnley
J.R. Isaacs
M.A. Quirin
B.A. McAllister