

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

FEBRUARY, 1983

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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## I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was compiled by Diane L. Lin, telephone number (815)357-6761, extension 499.

## II. SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE

February 1. The unit was shutdown due to the "B" RHR Pump being declared inoperative due to excessive pump vibration.

February 2-5. The reactor went critical at 1113 hours on February 2, 1983. At 2000 hours a steam leak was observed on a Main Steam Line Drain. At 2150 the reactor was shutdown. The reactor was critical for 10 hours and 37 minutes.

February 6-18. The reactor went critical at 1047 hours on February 6. At 0909 hours on February 7, 1983 the reactor was shutdown due to a steam leak on B Inboard Main Steam Line drain pipe. The reactor was critical for 22 hours and 22 minutes.

February 19-28. The reactor went critical at 0035 hours on February 19. At 2110 the main generator was synchronized to the grid and load increased to 100 MWe. At 1035 hours on February 20, 1983 the main turbine was unloaded for, LOS-TG-SA2, Turbine Valve Leak Tightness Surveillance. At 1810 hours the main generator was synchronized to the grid and loaded to 130 MWe. At 1500 hours on February 21, 1983 load was increased to 355 MWE. At 1545 hours on February 25, 1983 load was reduced to 200 MWe due to being unable to regenerate the condensate polishers. The reactor was critical for 239 hours and 25 minutes.

### III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE

#### A. Amendments to Facility License or Technical Specifications.

There were no amendment to facility license or technical specification during the reporting period.

#### B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

#### C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

#### D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

| WORK REQUEST | LER          | COMPONENT   | CAUSE OF MALFUNCTION                  | RESULTS AND EFFECTS<br>ON SAFE OPERATION                                 | CORRECTIVE ACTION               |
|--------------|--------------|---|---------------------------------------|--|---------------------------------|
| L20993       | ---          | 'B' RHR Outboard<br>Drain Valve                     | Broken torque switch                  | Indicates full closed when<br>valve is about 1 turn from<br>full closed. | Replaced torque<br>switch       |
| L21351       | ---          | 1B Diesel Generator<br>Air Compressor               | Bendix was bad                        | Would no start   | Replaced bendix                 |
| L21913       | ---          | CSCS Equipment<br>Cooling and Ventila-<br>tion Duct | Love controller out-of-<br>adjustment | Damper did not cycle   | Recalibrated love<br>controller |
| L22023       | ---          | IN Solenoid Valve                                   | Bad tape on lug connec-<br>tion       | Sparked when bumped  | Retaped lug connec-<br>tion     |
| L22145       | ---          | RCIC/LPCS Watertight<br>Door                        | Mechanism disconnected                | Interfers with proper door<br>operation                                  | Reconnected<br>mechanism        |
| L22270       | 83-008/03L-0 | Mechanical Snubber                                  | Missing pin in rear<br>bracket        | Snubber not pinned to<br>building  | Installed pin                   |
| L22331       | 83-007/03L-0 | 'C' Inboard MSIV<br>Drain Line                      | Crack in weld                         | Lines leaked   | Repaired crack                  |
| L22338       | ---          | 'D' Main Steam Line<br>Drain Valve                  | No lathern ring                       | Valve leaks  | Repacked valve                  |
| L22420       | 83-010/03L-0 | Piping Restraint                                    | Extra restraint                       | No effect  | Removed restraint               |
| L22453       | ---          | 1A MSIV Outboard<br>Drain Line                      | Bent line                             | No effect  | Replaced bent pipe              |
| L22454       | 83-001/01T-0 | RHR Injection Valve<br>Pressure Switch 'B'          | Bad switch                            | Switch leaked  | Replaced switch                 |
| L22455       | 83-001/01T-0 | RHR Injection Valve<br>Pressure Switch 'D'          | Bad switch                            | Nd effect  | Replaced switch                 |
| L22478       | ---          | VC B Train Radia-<br>tion Monitor                   | Bad amphenol connector                | Monitor will not stay<br>energized                                       | Replaced amphenol<br>connector  |
| L22489       | 83-001/01T-0 | 1B21-N413C Pressure<br>Switch                       | Bad switch                            | Switch leaked  | Replaced switch                 |

| WORK REQUEST     | LER | COMPONENT  | CAUSE OF MALFUNCTION  | RESULTS AND EFFECTS<br>ON SAFE OPERATION             | CORRECTIVE ACTION |
|------------------|-----|--|-----------------------|--|-------------------|
| L22490           | --- | Shutdown Cooling<br>Inboard Isolation<br>Valve   | Packing was loose     | Valve leaked   | Adjusted packing  |
| L22552           | --- | Bus 111Y Breaker #17                             | Water in junction box | Received ~30 V ground                                | Retaped leads     |
| L22578           | --- | 1A Diesel Generator<br>Relay K-11                | Defective diode       | No effect  | Replaced diode    |
| L22595           | --- | Main Steam Line<br>Inboard Drain Isolation Valve | Defective K-56 relay  | Valve does not close on a group one isolation signal | Replaced relay    |
| L22433<br>L22438 | --- | 1A Inboard MSIV<br>Drain Line                    | Bad weld              | Leaking  | Rewelded line     |
| L22385<br>L22386 | --- | 1B Inboard MSIV<br>Drain Line                    | Bad weld              | Leaking  | Rewelded line     |
| L22434<br>L22439 | --- | 1D Inboard MSIV<br>Drain Line                    | Bad weld              | Leaking  | Rewelded line     |
| L22411           | --- | 1A Outboard MSIV<br>Drain Line                   | Bad weld              | Leaking  | Rewelded line     |
| L22430<br>L22435 | --- | 1B Outboard MSIV<br>Drain Line                   | Bad weld              | Leaking  | Rewelded line     |
| L22431<br>L22436 | --- | 1C Outboard MSIV<br>Drain Line                   | Bad weld              | Leaking  | Rewelded line     |
| L22432<br>L22437 | --- | 1D Outboard MSIV<br>Drain Line                   | Bad weld              | Leaking  | Rewelded line     |

#### IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, February 1 to February 28, 1983. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

| <u>Licensee Event Report Number</u> | <u>Date</u> | <u>Title of Occurrence</u>   |
|-------------------------------------|-------------|--|
| 83-001/01T-0                        | 2/11/83     | Reactor Vessel Low Pressure LPCS/<br>LPCI Injection Valve Pressure |
| 83-005/01T-0                        | 2/7/83      | Leak Detection Temperature Switches<br>Out of Calibration          |
| 83-006/03L-0                        | 2/2/83      | Cracked Weld on MSL Drain  |
| 83-007/03L-0                        | 2/7/83      | Primary Containment Pressure<br>Boundary Leakage                   |
| 83-008/03L-0                        | /3/83       | Inoperable Mechanical Snubber                                      |
| 83-009/03L-0                        | 2/8/83      | Inoperable Mechanical Snubber                                      |
| 83-010/03L-0                        | 2/9/83      | Piping Restraint Installed in<br>Violation of Design               |



## V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

## OPERATING DATA REPORT

DOCKET NO. 050-373  
LaSalle  
 UNIT One  
 DATE March 3, 1983  
 COMPLETED BY Diane L. Lin  
 TELEPHONE (815)357-6761 x499

## OPERATING STATUS

1. REPORTING PERIOD: February 1983 GROSS HOURS IN REPORTING PERIOD: 672  
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX. DEPEND. CAPACITY (MW<sub>e</sub>-Net): 0  
 DESIGN ELECTRICAL RATING (MW<sub>e</sub>-Net): 1078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MW<sub>e</sub>-Net): \_\_\_\_\_

4. REASONS FOR RESTRICTION (IF ANY): \_\_\_\_\_

|  | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|------------|------------|------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL .....                  | 272.4      | 272.4      | 3019.8     |
| 6. REACTOR RESERVE SHUTDOWN HOURS .....                        | 0          | 0          | 0          |
| 7. HOURS GENERATOR ON LINE .....                               | 211.2      | 212.18*    | 2069.88*   |
| 8. UNIT RESERVE SHUTDOWN HOURS .....                           | 0          | 0          | 0          |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) .....                  | 224508     | 224508     | 2365087    |
| 10. GROSS ELECTRICAL ENERGY GENERATED (MWH) .....              | 58518      | 58518      | 578917     |
| 11. NET ELECTRICAL ENERGY GENERATED (MWH) .....                | 45934      | 45934      | 506709     |
| 12. REACTOR SERVICE FACTOR .....                               | NA         | NA         | NA         |
| 13. REACTOR AVAILABILITY FACTOR .....                          | NA         | NA         | NA         |
| 14. UNIT SERVICE FACTOR .....                                  | NA         | NA         | NA         |
| 15. UNIT AVAILABILITY FACTOR .....                             | NA         | NA         | NA         |
| 16. UNIT CAPACITY FACTOR (Using MDC) .....                     | NA         | NA         | NA         |
| 17. UNIT CAPACITY FACTOR (Using Design MW <sub>e</sub> ) ..... | NA         | NA         | NA         |
| 18. UNIT FORCED OUTAGE RATE .....                              | NA         | NA         | NA         |

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA

| 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): | FORECAST      | ACHIEVED       |
|---|---------------|----------------|
| INITIAL CRITICALITY                                       |               | <u>6/21/82</u> |
| INITIAL ELECTRICITY                                       |               | <u>9/4/82</u>  |
| COMMERCIAL OPERATION                                      | <u>4/1/83</u> |                |

\*Due to correction made in September 1982 report.

ATTACHMENT A  
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373  
UNIT LaSalle  
One  
DATE March 3, 1983  
COMPLETED BY Diane L. Lin  
TELEPHONE (815)357-6761 x499

MONTH February 1983

| DAY | AVERAGE DAILY POWER LEVEL<br>(MWe-Net) |
|-----|--|
| 1.  | <u>0</u>                               |
| 2.  | <u>0</u>                               |
| 3.  | <u>0</u>                               |
| 4.  | <u>0</u>                               |
| 5.  | <u>0</u>                               |
| 6.  | <u>0</u>                               |
| 7.  | <u>0</u>                               |
| 8.  | <u>0</u>                               |
| 9.  | <u>0</u>                               |
| 10. | <u>0</u>                               |
| 11. | <u>0</u>                               |
| 12. | <u>0</u>                               |
| 13. | <u>0</u>                               |
| 14. | <u>0</u>                               |
| 15. | <u>0</u>                               |
| 16. | <u>0</u>                               |

| DAY | AVERAGE DAILY POWER LEVEL<br>(MWe-Net) |
|-----|--|
| 17. | <u>0</u>                               |
| 18. | <u>0</u>                               |
| 19. | <u>0</u>                               |
| 20. | <u>64</u>                              |
| 21. | <u>265</u>                             |
| 22. | <u>360</u>                             |
| 23. | <u>391</u>                             |
| 24. | <u>341</u>                             |
| 25. | <u>261</u>                             |
| 26. | <u>182</u>                             |
| 27. | <u>167</u>                             |
| 28. | <u>177</u>                             |
| 29. | <u>---</u>                             |
| 30. | <u>---</u>                             |
| 31. | <u>---</u>                             |

ATTACHMENT B  
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February 1983

DOCKET NO. 050-373

UNIT NAME LaSalle One

DATE March 3, 1983

COMPLETED BY Diane L. Lin

TELEPHONE (815) 357-6761  
x499

| NO. | DATE      | TYPE<br>F: FORCED<br>S: SCHEDULED | DURATION<br>(HOURS) | REASON (1) | METHOD OF<br>SHUTTING DOWN<br>THE REACTOR OR<br>REDUCING POWER (2) | CORRECTIVE ACTIONS/COMMENTS   |
|-----|-----------|-----------------------------------|---------------------|------------|--|---|
|     |           |                                   |                     |            |  |   |
| 1   | 12/31/82* | F                                 | 45.8                | A          | 4  | Normal shutdown due to "Unusual Event" due to "B" RHR Pump being inop. because of high vibration. |
| 2   | 2/2/83    | F                                 | 107.3               | A          | 1  | Steam leak from "A" Out-board Main Steam Line Drain Pipe due to cracked welds.                    |
| 3   | 2/7/83    | F                                 | 300.1               | A          | 1  | Steam leak on "B" Inboard MSL drain pipe due to cracked weld.                                     |
| 4   | 2/20/83   | F                                 | 7.6                 | B          | 9  | Turbine off loaded to perform LOS-TG-SA2 Turbine Valve Leak Tightness Test.                       |

\*Refer to the December 1982 report.

## VI. UNIQUE REPORTING REQUIREMENTS

### A. Main Steam Relief Valve Operations for Unit 1

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was activated and the circumstances resulting in its actuation.

| <u>Date</u> | <u>Valves<br/>Actuated</u> | <u>No. &amp; Type<br/>Actuations</u> | <u>Plant<br/>Conditions</u> | <u>Description<br/>of Events</u>  |
|-------------|----------------------------|--------------------------------------|-----------------------------|---|
| 2/19/83     | 1B21-F013C                 | 2 Manual                             | 1000 psig                   | To verify proper valve operation after a solenoid replacement.  |
| 2/19/83     | 1B21-F013S                 | 3 Manual                             | 1000 psig                   | To verify proper position indication after removing the position indication assembly to gag the valve for a vessel hydro. |
| 2/19/83     | 1B21-F013U                 | 3 Manual                             | 1000 psig                   | To verify proper position indication after removing the position indication assembly to gag the valve for a vessel hydro. |

### B. ECCS Systems Outages

There were no ECCS System Outages during this reporting period.

### C. Off-Site Dose Calculation Manual.

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.

E. Process Control Program

There were no changes to the Process Control Program during this reporting period.