

TABLE 4.3.1.1-1
REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION ^(a)	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
1. Intermediate Range Monitors:				
a. Neutron Flux - High	S/U, S, ^(b) S	S/U, W W	R R	2 3, 4, 5
b. Inoperative	NA	W	NA	2, 3, 4, 5
2. Average Power Range Monitor: ^(f)				
a. Neutron Flux - High, Setdown	S/U, S, ^(b) S	S/U, W W	SA SA	2 3, 5
b. Flow Biased Simulated Thermal Power - High	S, D ^(b)	Q	W ^{(d)(e)} , SA, R ⁽ⁱ⁾	1
c. Neutron Flux - High	S	Q	W ^(d) , SA	1
d. Inoperative	NA	Q	NA	1, 2, 3, 5
3. Reactor Vessel Steam Dome Pressure - High	S	Q	R ^(g)	1, 2 ^(j)
4. Reactor Vessel Water Level - Low, Level 3	S	Q	R ^(g)	1, 2
5. Reactor Vessel Water Level - High, Level 8	S	Q	R ^(g)	1
6. Main Steam Line Isolation Valve - Closure	NA	Q	R	1
7. Main Steam Line Radiation - High	S	Q	R	1, 2 ^(j)
8. Drywell Pressure - High	S	Q	R ^(g)	1, 2 ^(k)

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NL-91/03

TABLE 4.3.1.1-1 (Continued)

REACTOR PROTECTION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL FUNCTIONAL TEST	CHANNEL CALIBRATION	OPERATIONAL CONDITIONS FOR WHICH SURVEILLANCE REQUIRED
9. Scram Discharge Volume Water Level - High				
a. Transmitter/Trip Unit	S	Q	R ^(g)	1, 2, 5 ⁽¹⁾
b. Float Switch	NA	Q	R	1, 2, 5 ⁽¹⁾
10. Turbine Stop Valve - Closure	S	Q	R ^(g)	1
11. Turbine Control Valve Fast Closure Valve Trip System Oil Pressure - Low	S	Q	R ^(g)	1
12. Reactor Mode Switch Shutdown Position	NA	R	NA	1, 2, 3, 4, 5
13. Manual Scram	NA	W	NA	1, 2, 3, 4, 5

(a) Neutron detectors may be excluded from CHANNEL CALIBRATION.

(b) The IRM and SRM channels shall be determined to overlap for at least 1/2 decade during each startup after entering OPERATIONAL CONDITION 2 and the IRM and APRM channels shall be determined to overlap for at least 1/2 decade during each controlled shutdown, if not performed within the previous 7 days.

(c) [DELETED]

(d) This calibration shall consist of the adjustment of the APRM channel to conform to the power values calculated by a heat balance during OPERATIONAL CONDITION 1 when THERMAL POWER > 25% of RATED THERMAL POWER. Adjust the APRM channel if the absolute difference is greater than 2% of RATED THERMAL POWER.

(e) This calibration shall consist of the adjustment of the APRM flow biased channel to conform to a calibrated flow signal.

(f) The LPRMs shall be calibrated at least once per 1000 MWD/T using the TIP system.

(g) Calibrate trip unit at least once per 92 days.

~~(h) Verify measured drive flow to be less than or equal to established drive flow at the existing flow control valve position.~~

(i) This calibration shall consist of verifying the 6 ± 1 second simulated thermal power time constant.

(j) Not applicable when the reactor pressure vessel head is unbolted or removed per Specification 3.10.1.

(k) Not applicable whe. DRYWELL INTEGRITY is not required.

(l) Applicable with any control rod withdrawn. Not applicable to control rods removed per Specification 3.9.10.1 or 3.9.10.2.

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NL-91/63