LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH, 1983

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station Unit One is a Boiling Water Reactor with a designed electrical output of 1078 MWe net, located in Marseilles, Illinois. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. The plant is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. The unit has not commenced commercial generation of power.

This report was complied by Diane L. Lin, telephone number (815)357-6761, extension 499.

II. SUMMARY OF UNIT OPERATING EXPERIENCE FOR UNIT ONE

March 1-12 The unit started the reporting period at approximately 190 MWe. At 1043 hours on March 12, 1983 the reactor scrammed on Hi Steam Tunnel ΔT MSIV Isolation due to VR being off and VQ on. The reactor was critical for 274 hours and 43 minutes.

March 12-17 The reactor went critical at 1905 hours on March 12, 1983. At 0250 hours on March 13, 1983 the main generator was synchronized to the grid and load increased to 12 MWe. At 1600 on March 14, 1983, load was increased to 637 MWe. At 0300 on March 17, 1983, power was decreased to 10%. At 0326 the turbine tripped on high vibrations due to low load. At 0750 the reactor scrammed due to a spurious signal from TSV fast closure. The reactor was critical for 108 hours and 45 minutes.

March 18-23 The reactor went critical at 1248 hours on March 18, 1983. At 2000 hours the main generator was synchronized to the gird and load increased to 200 MWe. At 0600 hours on March 19, 1983, load was increased to 400 MWe. At 1500 hours on March 20, 1983 load was increased to 710 MWe. The reactor scrammed on low level on March 22, 1983 at 0506 hours. The reactor was critical for 88 hours and 18 minutes.

March 24-31 The reactor went critical at 2012 hours on March 24, 1983. On March 25, 1983 at 0310 hours the main generator was synchronized to the grid and load increased. At 0000 hours on March 26, 1983 load was increased to 577 MWe. At 0350 load was decreased to 184 MWe. At 1230 hours on March 28, 1983 load was increased to 750 MWe. At 0830 on March 30, 1983 load was decreased to 477 MWe. At 2200 the reactor scrammed due to IRM B Hi alarm. The reactor was critical for 145 hours and 48 minutes.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications.

There were no amendment to facility license or technical specification during the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

C. Tests and Experiments Requiring NRC Approval.

There were no tests or experiments requiring NRC approval during the reporting period.

D. Corrective Maintenance of Safety Related Equipment.

The following tables present a summary of safety-related maintenance completed on Unit One during the reported period. The headings indicated in this summary include: Work Request Numbers, LER Numbers, Component Name, Cause of Malfunctions, Results and Effects on Safe Operation, and Corrective Action.

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L17534	-----	Reactor Water Instrument Stop Valve	Bad 3-Way Manifold	Valve will not seat when reactor pressure is ~200 PSI	Replaced 3-way manifold
L18545	-----	SBGT WRGM	Bad switch	Cannot operate switch	Replaced switch
L20289	-----	Channel A Turbine Control & Stop Valve Trip Bypass	Switches out-of-tolerance	Turbine trip gives ½ scram	Recalibrated switches
L22333	83-001/01T-0	Division I 125 Vdc Bus 111Y	Bad switch	Ground to 1E12-N009A	Replaced switch
L22347	83-013/03L-0	Leak Detection Temperature Switch	Bad switches	Ambient temperature switch and recorder have a 10° difference	Replaced 6 switches
L22379	83-009/03L-0	Mechanical Snubber	Bad snubber	Snubber hard to stroke	Replaced snubber
L22579	83-015/03L-0	Diesel Generator 1B Governor Control	Lead separated from coil	Oscillates on start-up	Resoldered lead to coil
L22730	-----	LPCS/LPCI Injection Valve Permissive	Bad switch	No effect	Replaced switch
L22742	-----	Core ΔP Recorder	Gear meshing and clutch out-of-adjustment	Recorder does not change with changes in core flow	Adjusted gear meshing and clutch
L22777	-----	'A' VC Charcoal Train Outlet Damper	Limit switch out-of-adjustment	Dual indication in Control Room	Adjusted limit switch
L22893	-----	Jet Pump #19 Flow Transmitter	Isolation valve leaking	Water leak on flow transmitter	Tightened nut on isolation valve
L22979	-----	Jet Pump Flow Indicator	Wrong head correction	Flow indicator indicated less than zero PSID	Recalculated head correction

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L23016	-----	Alarm A106	Bad annunciator input card	Alarm initiated erroneously	Replaced input card
L23310	-----	IRM F	Connector pin loose	IRM will not insert	Adjusted connector pin
L23359	-----	OPL58JB SEGT Analyzer	Bad PC board and vacuum transducer	Skid won't auto start	Repaired PC board and replaced vacuum transducer

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, March 1 to March 31, 1983. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in section 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

Licensee Event Report Number	Date	Title of Occurrence
83-011/03L-0	2-08-83	Rx. Vessel Low Water Level Scram and Primary Containment Isolation Instrumentation
83-012/03L-0	2-10-83	RCIC Pipe Routing Area High Temperature
83-013/03L-0	2-10-83	RWCU Area High Isolation
83-014/03L-0	2-15-83	Steam Line High Flow RCIC Isolation Instrument
83-015/03L-0	2-17-83	1B Diesel Generator Failure
83-016/03L-0	3-08-83	'0' and '1B' DG Room Ventilation Damper Electro-Thermal FP Links
83-017/03L-0	2-22-83	Failure of Lake Blowdown Flow Element 0FT-WL001
83-018/03L-0	2-26-83	RCIC Pump Suction Line High Pressure
83-019/03L-0	3-02-83	Lake Blowdown Flow Indication Broken
83-020/03L-0	3-05-83	Reactor Low Pressure RCIC Isolation

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

ATTACHMENT A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-373

UNIT LaSalle One

DATE April 7, 1983

COMPLETED BY Diane L. Lin

TELEPHONE (815) 357-6761 X499

MONTH March 1983

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	168	17.	10
2.	162	18.	5
3.	159	19.	464
4.	276	20.	648
5.	171	21.	663
6.	173	22.	127
7.	172	23.	0
8.	169	24.	0
9.	170	25.	311
10.	162	26.	322
11.	167	27.	627
12.	65	28.	684
13.	201	29.	678
14.	528	30.	274
15.	614	31.	0
16.	600		

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November 13, 1979
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ATTACHMENT B
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE April 7, 1983
COMPLETED BY Diane L. Lin
TELEPHONE (815) 357-6761 x499

REPORT MONTH March 1983

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
5	3/12/83	F	16.1	G	3	Rx Scram due to Hi Steam Tunnel ΔT MSIV Isolation due to VR off and VQ on.
6	3/17/83	F	0	B	5	To troubleshoot/repair B RR Flow Control Valve.
7	3/17/83	F	4.4	A	9	Turbine trip on high vibrations due to low load.
8	3/17/83	F	36.2	A	3	Rx Scram due to spurious signal from TSV fast closure
9	3/22/83	F	70.1	A	3	Rx Scram on low level.
10	3/26/83	F	0	A	5	Due to 1C71-N006C not functioning properly.
11	3/30/83	F	0	B	5	To remove CP's from line and to isolate 1/2 of the condenser to look for leaks.
12	3/30/83	F	26.8	A	3	Rx Scram due to IRM B Hi Alarm.

ATTACHMENT D

LTP-300-7
Revision 2
November 15, 1979
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OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE April 7, 1983
COMPLETED BY Diane L. Lin
TELEPHONE (815) 357-6761 X499

OPERATING STATUS

1. REPORTING PERIOD: March 1983 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 100% MAX. DEPEND. CAPACITY (MWe-Net): 0
   DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): _____
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	617.6	890	4527.4
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	590.5	802.68	2660.38
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	794333	1018841	3159420
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	226534	285052	805451
11. NET ELECTRICAL ENERGY GENERATED (MWH)	209033	254967	715742
12. REACTOR SERVICE FACTOR	NA	NA	NA
13. REACTOR AVAILABILITY FACTOR	NA	NA	NA
14. UNIT SERVICE FACTOR	NA	NA	NA
15. UNIT AVAILABILITY FACTOR	NA	NA	NA
16. UNIT CAPACITY FACTOR (Using MDC)	NA	NA	NA
17. UNIT CAPACITY FACTOR (Using Design MWe)	NA	NA	NA
18. UNIT FORCED OUTAGE RATE	NA	NA	NA

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 4-6-83

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	6-21-82
INITIAL ELECTRICITY	_____	9-04-82
COMMERCIAL OPERATION	6-01-83	_____

VI. UNIQUE REPORTING REQUIREMENTS

A. Main Steam Relief Valve Operations for Unit 1

There were no main steam relief valve operations for the reporting period.

B. ECCS Systems Outages

There were no ECCS System Outages during this reporting period.

C. Off-Site Dose Calculation Manual

There were no changes to the Off-Site Dose Calculations Manual during this reporting period.

D. Radioactive Waste Treatment System

There were no changes to the Radioactive Waste Treatment System during this reporting period.

E. Process Control Program

There were no changes to the Process Control Program during this reporting period.