



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

April 23, 1991

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Byron Station Units 1 and 2
Braidwood Station Units 1 and 2
Generic Letter 88-17 Supplemental Information
Loss of Decay Heat Removal
TAC Nos. 69725, 69726, 69727
NRC Docket Nos. 50-454/455 and 50-456/457

Reference: a) January 22, 1990 letter from
T.K. Schuster to T.E. Murley

Dear Dr. Murley:

Commonwealth Edison provided status on the implementation of Generic Letter 88-17, Loss of Decay Heat Removal, for Byron and Braidwood Stations in reference (b). This letter provides an update on the status of the Residual Heat removal (RHR) system iconic display at Byron and Braidwood Stations.

Byron Station staff has designed a computer graphic for the Main Control Room. The graphic displays data in real time and reflects the status of the refueling water level and RHR pump parameters. The display is a mimic of the reactor vessel, steam generator, pressurizer and RCP, and is to scale with respect to plant elevation. The screen displays bar graphs and literal values for the four refueling level instruments and a pressurizer level instrument. Also shown are the RHR pump suction and discharge pressures, discharge temperature and motor bearing temperatures.

This display software installation has been completed at Byron Station. Braidwood Station was scheduled to install the display software on both units by January 15, 1991, however, this modification has not yet been completed. When the software was installed it would not load properly and was returned to the computer group for revision. The installation of this modification is expected to be completed by July 1, 1991. Please direct any further questions regarding this matter to this office.

Very truly yours,

Allen R. Checca
Nuclear Licensing Administrator

ZNLD/897

cc: Resident Inspector - Byron
Resident Inspector - Braidwood
A.H. Hsia - NRR
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