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April 6, 1983

Docket No. 50-278

Mr. John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Mr. Stolz:

The purpose of this letter is to provide additional information and to modify that portion of our request for amendment of Facility Operating License DPR-56 submitted on December 30, 1982, pertaining to continued operation of the reconstituted Pressurized Test Assembly (PTA).

An analysis of the safety considerations involved in continued operation of the reconstituted PTA was provided in a document titled "Supplemental Reload Licensing Submittal for Peach Bottom Atomic Power Station Unit 3, Reload 5" (Y1003J01A54, December, 1982) which was filed with the Amendment Application of December 30, 1982. The program plan described in Y1003J01A54 included the removal of twenty PTA fuel rods for fission gas measurements and replacement of these PTA fuel rods with irradiated fuel rods from a discharged 8DRB283 type design fuel assembly. This plan was modified during the on-site inspection to include the replacement of two additional PTA fuel rods when potential damage occurred during the inspection of a PTA fuel rod which was not originally scheduled for replacement. Subsequent examination of the fuel rod in question indicated no apparent damage. However, as a precautionary measure, the fuel rod and its diagonally symmetric counterpart were removed from the PTA and exchanged with two additional irradiated rods from the same 8DRB283 fuel assembly.

The effects of replacing twenty-two PTA fuel rods rather than the original twenty have been evaluated by General Electric and it is concluded that the safety evaluations and analyses provided in document Y1003J01A54 are unchanged. The re-analysis of the PTA reconstituted with twenty-two fuel rods demonstrate that the Operating Limit Minimum Critical Power Ratio (MCPR)

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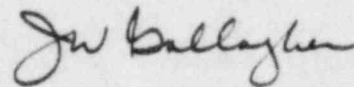
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values previously submitted remain applicable. Peak cladding temperature will remain below 2200 degrees F and the Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) values, previously submitted on December 30, 1982 in document "LOCA Analysis for Peach Bottom Atomic Power Station Unit 3", Errata and Addenda Sheet No. 5, dated October 1982 remain applicable. In addition, the results of the fuel rod thermal and mechanical design evaluations contained in document NEDO-21363-4, Supplement 4, dated January 1977, previously submitted, remain conservatively applicable for the PTA reconstituted with twenty-two fuel rods.

Based on the foregoing, it is requested that continued operation of the PTA be permitted as reconstituted with twenty-two fuel rods.

Should you have any questions regarding this matter, please do not hesitate to contact us.

Very truly yours,



cc: G. E. Gears, Project Manager
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U.S. Nuclear Regulatory Commission
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A. R. Blough, Site Inspector
Peach Bottom