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September 13, 1994
C311-94-2130

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for August 1994

Enclosed are two copies of the August 1994 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton
Vice President and Director, TMI

WGH

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector
T94001

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R PDR

GPU Nuclear Corporation is a subsidiary of General Public Utilities Corporation

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OPERATIONS SUMMARY

August 1964

The plant entered the month operating at 100% power and remained essentially at that level through 1458 hours on August 31. At that time a power reduction was initiated to replace a faulty Main Turbine Electro-Hydraulic Control (EHC) Load Reference Card. The unit was taken off line at 1658 and the turbine work was performed while the reactor plant was maintained at 5-10% power. Following the repair, the unit was brought back on line at 1843 and power was gradually escalated to 100%. Unit electrical output averaged approximately 786 MWe during August.

MAJOR SAFETY RELATED MAINTENANCE

The following is a summary of major safety related maintenance items accomplished during the month.

NR-V-13D Piping

A pin hole leak in the 12" pipe upstream of Nuclear Service Closed Cooler NS-C-1D backwash valve NR-V-13D was repaired by welding a half coupling over the hole. The weld was satisfactorily tested using the magnetic particle process prior to returning the system to service.

Intermediate Closed Cooling Water Pump IC-P-1A

Intermediate Closed Cooling Water pump IC-P-1A was removed from service to replace a leaking mechanical seal. IC-P-1A was retested satisfactorily and returned to service. Later in the month, shaft leakage was observed and is being monitored.

Beckman Analyzer CA-G-1A

A failed pump diaphragm caused the removal of Beckman Analyzer CA-G-1A from service. The pump was disassembled and rebuilt using new parts. Post maintenance testing was completed satisfactorily and the pump was returned to service.

Main Turbine Electro-Hydraulic Control System

On August 31 at 1458 hours, a power reduction was initiated for the purpose of replacing a failed Main Turbine Electro-Hydraulic Control (EHC) system computer card. The card provides a load control reference signal and its output had failed low causing plant oscillation. At 1658 hours, the breakers were opened, the turbine taken off-line, and the EHC card replaced. The breakers were closed at 1843 hours and the plant returned to 100% power at 0255 hours on September 1.

Integrated Control System

With the turbine plant off-line for EHC repairs, an electronic module in the Plant Integrated Control System (ICS) was replaced. The ICS developed a problem in the 'B' Feedwater Control Loop which was attributed to the ICS module.

OPERATING DATA REPORT

OPERATING STATUS

DOCKET NO. 50-289
 DATE September 13, 1994
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

1. UNIT NAME: THREE MILE ISLAND UNIT 1
 2. REPORTING PERIOD: AUGUST 1994
 3. LICENSED THERMAL POWER: 2568
 4. NAMEPLATE RATING (GROSS MWe): 871
 5. DESIGN ELECTRICAL RATING (NET MWe): 819
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT. GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	5831.0	175296.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	5433.7	98659.7
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	-0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	742.3	5421.4	97526.1
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1891486	13677373	238544467
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	619264	4536463	80223246
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	584514	4267907	75332426
19. UNIT SERVICE FACTOR	(%)	99.8	93.0	55.6
20. UNIT AVAILABILITY FACTOR	(%)	99.8	93.0	55.6
21. UNIT CAPACITY FACTOR (USING MDC NET)		100.0	93.1	54.7
22. UNIT CAPACITY FACTOR (USING DER NET)		95.9	89.4	52.5
23. UNIT FORCED OUTAGE RATE	(%)	0.2	0.0	38.3
UNIT FORCED OUTAGE HOURS	(HRS)	1.8	1.8	60761.2
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE September 13, 1994
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: AUGUST

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
-----	--

1	787
2	786
3	789
4	786
5	793
6	806
7	800
8	799
9	793
10	794
11	797
12	790
13	783
14	785
15	801
16	800

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
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17	794
18	789
19	792
20	789
21	788
22	795
23	803
24	802
25	797
26	794
27	790
28	774
29	792
30	786
31	582

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-289
UNIT NAME TWI-1
DATE September 13, 1994
COMPLETED BY W. G. Heysek
TELEPHONE (717) 948-8191

REPORT MONTH August 1994

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ²	Licensing Event Report#	System Code ^{3, 4}	Component Code ^{3, 4}	Cause & Corrective Action to Prevent Recurrence
94-06	08/31/94	F	1.8	A	4	None	IT	AMP	A power reduction was initiated at 1500 hours in order to replace a faulty EHC Load Reference card. The unit was taken off line at 1658 and the reactor maintained at 5-10% power while the work was accomplished. Following the completion of the card replacement, the unit was brought back on line at 1843 and gradually escalated to 100% power.

1
F Forced
S Scheduled

2
Reason
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restrictions
E-Operator Training & Licensing Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit G - Instructions for preparation of Data Entry Sheets for Licensing Event Report (LER) File (NUREG-0161)

5 Exhibit 1 same source

6 Actually used exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: **Three Mile Island Nuclear Station, Unit 1**
2. Scheduled date for next refueling shutdown: **September 8, 1995**
3. Scheduled date for restart following current refueling: **NA**
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? **NO**
5. Scheduled date(s) for submitting proposed licensing action and supporting information: **NA**
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
 - a) TMI will use the new Mark B10 fuel assembly in the Cycle 11 reload batch which is an upgraded design of the Mark B9 assembly used in Cycle 10. The Mark B10 provides a leaf-type cruciform assembly holddown spring to replace the previous coil spring design which has experienced random failures during operation and requires visual inspection each outage. The Mark B10 design meets all current BWFC fuel design criteria and is in use at other B&W 177 FA plants.
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 601
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.