



Boston Edison

Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier

Vice President Nuclear Operations
and Station Director

September 14, 1994
BECO Ltr. #94-103

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

AUGUST 1994 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.


L.J. Olivier

WJM/lam/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
 DATE 9/14/94
 COMPLETED BY: W. Munro
 TELEPHONE (508) 830-8474

OPERATING STATUS

NOTES

- | | |
|--|-------------|
| 1. Unit Name | Pilgrim I |
| 2. Reporting Period | August 1994 |
| 3. Licensed Thermal Power (MWt) | <u>1998</u> |
| 4. Nameplate Rating (Gross MWe) | <u>678</u> |
| 5. Design Electrical Rating (Net MWe) | <u>655</u> |
| 6. Maximum Dependable Capacity (Gross MWe) | <u>696</u> |
| 7. Maximum Dependable Capacity (Net MWe) | <u>670</u> |
| 8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons: | |

NONE

9. Power Level To Which Restricted, If Any (Net MWe): None
 10. Reasons For Restrictions, If Any: N/A

This Month Yr-to-Date Cumulative

- | | | | |
|--|--|-------------------|--------------------|
| 11. Hours in Reporting Period | <u>744.0</u> | <u>5831.0</u> | <u>190463.0</u> |
| 12. Hours Reactor Critical | <u>679.5</u> | <u>5473.5</u> | <u>118415.0</u> |
| 13. Hours Reactor Reserve Shutdown | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 14. Hours Generator On-Line | <u>679.5</u> | <u>5354.3</u> | <u>114143.3</u> |
| 15. Hours Unit Reserve Shutdown | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 16. Gross Thermal Energy Generated (MWH) | <u>1341816.0</u> | <u>10275000.0</u> | <u>201419736.0</u> |
| 17. Gross Electrical Energy Generated (MWH) | <u>456930.0</u> | <u>3518470.0</u> | <u>68164404.0</u> |
| 18. Net Electrical Energy Generated (MWH) | <u>439687.0</u> | <u>3386209.0</u> | <u>65519903.0</u> |
| 19. Unit Service Factor | <u>91.3</u> | <u>91.8</u> | <u>59.9</u> |
| 20. Unit Availability Factor | <u>91.3</u> | <u>91.8</u> | <u>59.9</u> |
| 21. Unit Capacity Factor (Using MDC Net) | <u>88.2</u> | <u>86.7</u> | <u>51.3</u> |
| 22. Unit Capacity Factor (Using DER Net) | <u>90.2</u> | <u>88.7</u> | <u>52.5</u> |
| 23. Unit Forced Outage Rate | <u>8.7</u> | <u>4.6</u> | <u>11.5</u> |
| 24. Shutdowns Scheduled Over Next 6 Months
(Type, Date, and Duration of Each) | MCO-10 September 1994 | | |
| 25. If Shutdown at End of Report Period,
Estimated Date of Startup - | Unit Shutdown, Estimated Date of Startup to be determined. | | |

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	<u>50-293</u>
DATE:	<u>9/14/94</u>
COMPLETED BY:	<u>W. Munro</u>
TELEPHONE:	<u>(508) 830-8474</u>

MONTH August 1994

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	658	17	655
2	656	18	656
3	656	19	653
4	658	20	652
5	659	21	657
6	655	22	654
7	655	23	629
8	655	24	650
9	653	25	652
10	655	26	651
11	654	27	640
12	654	28	478
13	656	29	203
14	657	30	0
15	656	31	0
16	655		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR AUGUST 1994

The plant started the reporting period at 100 percent core thermal power (CTP) where it was essentially maintained until August 28, 1994. During this period, on August 3, 1994, at 1122 hours, an automatic Primary Containment Isolation Control System (PCIS) Group 5 actuation occurred during performance of the Reactor Core Isolation Cooling System (RCIC) quarterly surveillance test, causing the RCIC system to be declared inoperable. Following maintenance, the RCIC system was returned to operable status on August 12, 1994.

On August 27, 1994, the plant commenced a power reduction to approximately 46 percent CTP to facilitate a main condenser backwash. Following the backwash reactor power was increased to 100 percent CTP. On August 29, 1994, at 0730 hours the main turbine generator tripped off line due to a load reject, causing an automatic reactor scram. During the scram, increased reactor pressure caused three (3) of the safety relief valves to open momentarily, and then reset. At 0937 hours the reactor mode switch was placed in the shutdown position. The reactor was in shutdown cooling at 2033 hours where it remained to the end of the reporting period.

During the reporting period unusually high temperature inlet water and condenser fouling adversely affected the main condenser back pressure. On various occasions, it was necessary to briefly trim reactor power to maintain adequate main condenser vacuum.

A Multi Discipline Assessment Team (MDAT) was formed to evaluate the cause of the load reject and the reactor scram.

SAFETY RELIEF VALVE CHALLENGES
MONTH OF AUGUST 1994

Requirement: NUREG-0737 T.A.P. II.K.3.3

Date: August 29, 1994

Valve #203-3A, 3B, and 3C

Reason: Generator load reject - Reactor Scram

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure, auto signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.
2. Scheduled date for next refueling shutdown: April 1, 1995.
3. Scheduled date for restart following next refueling: May 26, 1995.
4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.
5. See #6.
6. The new fuel loaded during the 1993 refueling outage was of the same design as loaded in the previous refueling outage and consisted of 140 assemblies.
7.
 - (a) There are 580 fuel assemblies in the core.
 - (b) There are 1629 fuel assemblies in the spent fuel pool.
8.
 - (a) The station is presently licensed to store 3859 spent fuel assemblies. The actual usable spent fuel storage capacity is 2320 fuel assemblies.
 - (b) The planned spent fuel storage capacity is 3859 fuel assemblies.
9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 691 fuel assemblies.

MONTH AUGUST 1994

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER
Reactor Core Isolation Cooling (RCIC) System	RCIC Turbine Steam Governor Control Valve HO-1301-159	During performance of RCIC Surveillance Procedure 8.5.5.1 an automatic Primary Containment Isolation Control System Gp 5 actuation occurred due to high steam flow signal. RCIC System was declared inoperable. (PR94.9313) (PR94.9413)	Binding of the RCIC Governor Control Valve (HO-1301-159) due to misalignment of the control valve fulcrum alignment pins. Root cause is under investigation.	The Governor Control Valve was rebuilt, properly aligned and successfully tested.	Refer to associated LER.	94-004-00
High Pressure Coolant Injection (HPCI) System	HPCI Turbine Exhaust Pressure Indicator PI-2367	During replacement of PI-2367, discovered instrument snubber line clogged. (PR94.9312)	Excess thread sealant in instrument snubber. Root cause under investigation.	Replaced clogged snubber.	To be determined.	N/A

UNIT SHUTDOWNS AND POWER REDUCTIONS DOCKET NO: 50-293

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NAME: Pilgrim I
DATE: 9/14/94
COMPLETED BY: W. Munro
TELEPHONE: (508) 830-8474
REPORT MONTH: August 1994

NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
8	8/28/94	S	0.0	B	N/A	N/A	N/A	N/A	Power reduction to facilitate backwash of the main condenser.
9	8/29/94	F	64.5	A	3	94-005	TB, TJ	GEN	Automatic scram due to load reject

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)