NRC FORM 366 (12-81) 10 CFR 50

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB 3150-0011

LICENSEE EVENT REPORT

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | LICENSEE CODE: P A S E S I ② | LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 ③ | LICENSE TYPE: 4 1 1 1 1 ④ | CAT ⑤

CON'T

0 1 | REPORT SOURCE: L ⑥ | DOCKET NUMBER: 0 5 0 0 0 3 8 7 ⑦ | EVENT DATE: 0 2 2 8 8 3 ⑧ | REPORT DATE: 0 3 3 0 8 3 ⑨

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 While the plant was at 78% power, a 4 hour surveillance of reactor coolant system
0 3 unidentified leakage could not be performed (Technical Specification 3.4.3.2.e)
0 4 due to temporary inoperability of one of the two drywell floor drain sump level
0 5 monitoring system channels (Technical Specification 3.4.3.1.b). There were no
0 6 consequential effects to the public health and safety. There is relative assur-
0 7 ance that the 4 hour leakage was not greater than 0.4gpm.
0 8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	C I ⑪	X ⑫	Z ⑬	Z Z Z Z Z Z ⑭	Z ⑮	Z ⑯

⑰ LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 3	—	0 3 5	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS ㉒	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER ㉖
X ⑱	X ⑲	Z ⑳	Z ㉑	0 0 0 0	Y ㉓	N ㉔	Z ㉕	Z 9 9 9

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

1 0 The'B'pump in the'A' drywell sump came on and pumped the sump down to 25% level.
1 1 For the next 4 hrs.the indicated sump level did not change,but the pump continued
1 2 running. When the sump level problem was observed,the'B'pump was shut down & the
1 3 'A'pump was used to pump the sump. Leakage calculations were run prior to & after
1 4 the 4 hrs.period in question and leakage rates were 0.38 & 0.4gpm.

1 5 | FACILITY STATUS: B ㉘ | % POWER: 0 7 8 ㉙ | OTHER STATUS ㉚: NA | METHOD OF DISCOVERY: B ㉛ | DISCOVERY DESCRIPTION ㉜: Operator observation

1 6 | ACTIVITY RELEASED: Z ㉝ | CONTENT OF RELEASE: Z ㉞ | AMOUNT OF ACTIVITY ㉟: NA | LOCATION OF RELEASE ㊱: NA

PERSONNEL EXPOSURES

1 7 | NUMBER: 0 0 0 ㊲ | TYPE: Z ㊳ | DESCRIPTION ㊴: NA

PERSONNEL INJURIES

1 8 | NUMBER: 0 0 0 ㊵ | DESCRIPTION ㊶: NA

LOSS OF OR DAMAGE TO FACILITY ㊸

1 9 | TYPE: Z ㊷ | DESCRIPTION: NA

PUBLICITY

2 0 | ISSUED: N ㊹ | DESCRIPTION ㊺: NA

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PDR ADOCK 05000387
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NRC USE ONLY

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Attachment

Licensee Event Report 83-035/03L-0

The reactor coolant system unidentified leakage is calculated from the drywell floor drain sumps. There are two pumps in each sump. Each sump has one level transmitter that feeds one level recorder. The calculations are made from the last time the calculation was performed until the present. For each calculation, the total percent that the sumps were pumped down is determined and then multiplied by a constant which converts this total percent to gallons. The total gallons are then divided by the total minutes from the last time the calculation was performed and this results in the average in-leakage.

The reason for the level transmitter failure to indicate less than 25% level when the 'B' pump was running is not entirely clear. The problem does not exist when the 'A' pump is used. Until the next extended outage when the problem can be further investigated, the 'A' pump will be used exclusively for leakage indication.