



Commonwealth Edison
1400 Opus Place
Downers Grove, Illinois 60515

April 8, 1991

Dr. Thomas E. Murley, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Byron Station Units 1 and 2
Braidwood Station Units 1 and 2
Amendment of Facility Operating Licenses
NPF-37, NPF-66, NPF-72, and NPF-71
NRC Docket Nos. 50-454/455 and 50-455/456
TAC Nos. M69527, 69528, 69529, 69530

- Reference: (a) October 4, 1988 letter from R.A. Chrzanowski to T.E. Murley.
- (b) August 14, 1989 letter from R.A. Chrzanowski to T.E. Murley.
- (c) March 20, 1991 letter from A.R. Checca to T.E. Murley.

Dear Dr. Murley

Reference (a) provided the request for amendment to the Byron and Braidwood Station Technical Specifications to permit the use of continuous pressure testing of the containment airlock door seal gaskets in lieu of manual testing. Reference (b) provided information about the radiation exposure reduction that would occur with this amendment. After discussions with the staff it was determined that additional surveillance testing of the containment airlock seals would increase the assurance that the limiting condition for operation would be met. Consequently, a revised technical specification page was submitted in reference (c).

Through an administrative oversight, reference (c) did not include the revised page for the Byron Technical Specifications. The revised Byron Technical Specification page is contained in the enclosure. The proposed change has been reviewed and approved by both on-site and off-site review in accordance with Commonwealth Edison procedures. Commonwealth Edison has reviewed this proposed amendment in accordance with 10 CFR 50.59(c) and has determined that no significant hazard consideration exists. This evaluation was previously submitted in reference (a).

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