



Nebraska Public Power District

COOPER NUCLEAR STATION
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321
TELEPHONE (402)825-3811
FAX (402)825-5211

CNSS948299

September 8, 1994

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Cooper Nuclear Station Licensee Event Report 94-017 is forwarded as an attachment to this letter.

Sincerely,

R. L. Gardner
Plant Manager

RLG/nc

Attachment

cc: L. J. Callan
G. R. Horn
J. H. Mueller
S. J. Jobe
R. A. Sessoms
R. E. Wilbur
D. A. Whitman
INPO Records Center
NRC Resident Inspector
R. J. Singer
CNS Training
CNS Quality Assurance

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NRC FORM 366 (5-92)		U.S. NUCLEAR REGULATORY COMMISSION			APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95					
LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)										
FACILITY NAME (1) COOPER NUCLEAR STATION					DOCKET NUMBER (2) 05000298		PAGE (3) 1 OF 1			
TITLE (4) Potential Breach of Primary Containment Through the Reactor Equipment Cooling System Due to Rupture of System Piping Inside the Drywell Caused by a HELB										
EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	09	94	94	-- 017 --	00	09	08	94	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)						
POWER LEVEL (10)		0		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)
				20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)
				20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER
				20.405(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 366A)
				20.405(a)(1)(iv)		X 50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
				20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)		
LICENSEE CONTACT FOR THIS LER (12)										
NAME Donald L. Reeves, Jr.						TELEPHONE NUMBER (Include Area Code) (402) 825-3811				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
X	YES (If yes, complete EXPECTED SUBMISSION DATE).				NO			10	14	94
ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16) <p>A re-assessment of the condition described in IN 89-55 and the ensuing engineering evaluation performed in 1989 raised concerns that the evaluation may not have been responsive to the intent or the potential ramifications of the condition addressed by the IN. IN 89-55 is directly applicable to the Reactor Equipment Cooling (REC) System in that it is the only closed system utilizing the specific isolation scheme described. The scenario described in the IN is of concern because a vent path from containment through the REC Drywell return line and out the REC Surge Tank 4" vent into the Reactor Building (Secondary Containment) could develop.</p> <p>The original assessment of the condition described in IN 89-55 concluded that the concern was not applicable since the probability of a Reactor Recirculation System line break causing an REC System line break inside containment together with a subsequent failure of an REC Containment Isolation Valve was very remote.</p> <p>An evaluation of the re-assessment is ongoing. Prior to plant startup, this concern will be resolved and a supplement to this LER will be submitted.</p>										