

Arizona Public Service Company

P.O. BOX 21666 • PHOENIX, ARIZONA 85036

March 17, 1983

ANPP-23271-RQT/BSK

1983 MAR 24 PM 12:53

U. S. Nuclear Regulatory Commission
Region V
Creskide Oaks Office Park
1450 Maria Lane - Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. D. M. Sternberg, Chief
Reactor Projects Branch 1

Subject: Interim Report - DER 83-10
A 50.55(e) Potentially Reportable Deficiency Relating to
Main Steam Relief Valve Tests Exceed Specified Five Percent
(5%) Blowdown Limit
File: 83-019-026
D.4.33.2

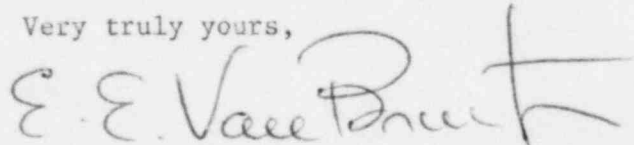
Reference: Telephone Conversation between P. Narbut and
G. Duckworth on February 28, 1983

Dear Sir:

The NRC was notified of a potentially reportable deficiency in the
referenced telephone conversation. At that time, it was estimated that a
determination of reportability would be made within thirty (30) days.

Due to the extensive investigation and evaluation required, an Interim
Report is attached. It is now expected that this information will be
finalized by May 24, 1983, at which time a complete report will be
submitted.

Very truly yours,



E. E. Van Brunt, Jr.
APS Vice President
Nuclear Projects Management
ANPP Project Director

EEVBJr/RQT:db

Enclosure

cc: See Attached Page 2

1927

U. S. Nuclear Regulatory Commission
Attention: Mr. D. M. Sternberg, Chief
Page 2

March 17, 1983
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cc: Richard DeYoung, Director
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

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INTERIM REPORT - DER 83-10
POTENTIAL REPORTABLE DEFICIENCY
ARIZONA PUBLIC SERVICE COMPANY (APS)
PVNGS UNITS 1, 2 & 3

I. POTENTIAL PROBLEM

The Unit 1 main steam safety valves had been received and installed. The valves were removed for Secondary Hydro Testing and stored in violation of ANSI N45.2.2 and Dresser Installation/Maintenance Manuals. A build-up of corrosion on parts requiring close tolerances, and the potential misalignment of valve internals, precluded valve operation per their design intent. To assure valve performance to design intent, APS requested the valves be transmitted to Wyle Laboratories where the boiler capacity existed for full flow testing. Prior to testing, the valves were cleaned and refurbished as required.

Ten (10) of the twenty (20) relief valves were tested, and all valves exceeded the specified blowdown of five percent (5%). Blowdown for the tested valves ranged from seven percent (7%) to fifteen percent (15%). Increased blowdown results in higher steam releases, increased cooldown rates of primary and secondary systems and potentially higher offsite radiological doses. The aforementioned valves are Dresser Model No. 370RA-RT25. The following Tag Nos. are affected:

SGE-PSV-554	SGE-PSV-559	SGE-PSV-574	SGE-PSV-579
SGE-PSV-555	SGE-PSV-560	SGE-PSV-575	SGE-PSV-591
SGE-PSV-556	SGE-PSV-561	SGE-PSV-576	SGE-PSV-592
SGE-PSV-557	SGE-PSV-572	SGE-PSV-577	SGE-PSV-594
SGE-PSV-558	SGE-PSV-573	SGE-PSV-578	SGE-PSV-595

II. APPROACH TO AND STATUS OF PROPOSED RESOLUTION

Bechtel Engineering is currently corresponding with Combustion Engineering to disposition this item.

III. PROJECTED COMPLETION OF CORRECTIVE ACTION
AND SUBMITTAL OF THE FINAL REPORT

Evaluation of this condition and submittal of the Final Report is forecast to be completed by May 24, 1983.