

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: 1 2 3 4 5 6

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

REPORT SOURCE 60 DOCKET NUMBER 68 EVENT DATE 74 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

On 6/9/83, during a reactor trip recovery, the auxiliary feedwater pump P-75 became inoperable due to a break in the seal supply piping which resulted in subsequent seal failure. The loss of P-75 caused entry into an action statement as required by Technical Specification (TS) 3.4.3. Both emergency feedwater pumps were inoperable and available to feed the steam generators. This occurrence is reportable per T.S. 6.12.3.2.b.

SYSTEM CODE 11 CAUSE CODE 12 CAUSE SUBCODE 13 COMPONENT CODE 14 COMP SUBCODE 15 VALVE SUBCODE 16

LER/RO REPORT NUMBER 17 EVENT YEAR 21 22 SEQUENTIAL REPORT NO. 24 OCCURRENCE CODE 28 REPORT TYPE 30 REVISION NO. 32

ACTION TAKEN 33 FUTURE ACTION 34 EFFECT ON PLANT 35 SHUTDOWN METHOD 36 HOURS 37 ATTACHMENT SUBMITTED 41 NPRD-4 FORM SUB 42 PRIME COMP. SUPPLIER 43 COMPONENT MANUFACTURER 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

The cause of the seal supply piping and subsequent seal failure can be attributed to misalignment of the seal supply piping. Immediate action was to lower secondary pressure until the steam generators could be fed with the condensate pumps to secure P-75. It was decided not to use emergency feedwater. The broken portion of the piping was replaced, and the piping was realigned to reduce stresses. The pump was tested satisfactorily and returned to service.

FACILITY

STATUS 9 % POWER 10 OTHER STATUS 13 METHOD OF DISCOVERY 30 DISCOVERY DESCRIPTION 32

ACTIVITY

RELEASED 9 CONTENT 10 AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

PERSONNEL EXPOSURES

NUMBER 9 TYPE 10 DESCRIPTION 11

PERSONNEL INJURIES

NUMBER 9 DESCRIPTION 10

LOSS OF OR DAMAGE TO FACILITY

TYPE 9 DESCRIPTION 10

PUBLICITY

ISSUED 9 DESCRIPTION 10

NAME OF PREPARER

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PDR ADOCK 05000313
S PDR

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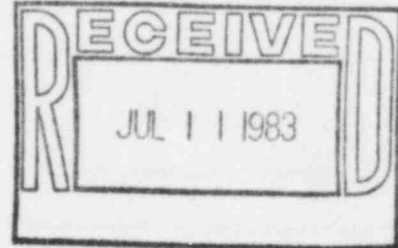


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July 5, 1983

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Mr. W. C. Seidle, Chief
Reactor Project Branch #2
U. S. Nuclear Regulatory Commission
Region IV
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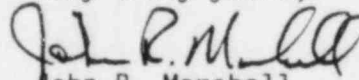


Subject: Arkansas Nuclear One - Unit 1
Docket No. 50-313
License No. DPR-51
Licensee Event Report
No. 83-014/03L-0

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 1 Technical Specification 6.12.3.2.b, attached is the subject report concerning an auxiliary feedwater pump seal failure

Very truly yours,


John R. Marshall
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard C. DeYoung
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U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
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