



South Carolina Electric & Gas Company
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Refer to: RC-94-0227

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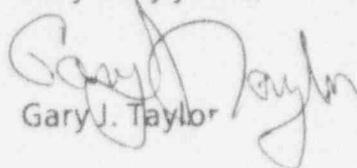
Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION
DOCKET NO. 50/395
OPERATING LICENSE NO. NPF-12
SEMI-ANNUAL EFFLUENT AND WASTE
DISPOSAL REPORT (SA 300)

Enclosed is the South Carolina Electric & Gas Company (SCE&G) Semi-Annual Effluent and Waste Disposal Report as required by 10CFR50.36a, Sections 6.9.1.8 and 6.14.2 of the Virgil C. Summer Nuclear Station Technical Specifications, and Section 1.6.2 of the Offsite Dose Calculation Manual (ODCM).

Should there be any questions, please contact us at your convenience.

Very truly yours,


Gary J. Taylor

SBR:lcd
Enclosure

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SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

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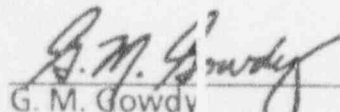
January 1, 1994 - June 30, 1994

August, 1994

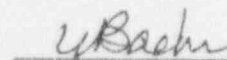


V. C. SUMMER NUCLEAR STATION
SOUTH CAROLINA ELECTRIC AND GAS COMPANY

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SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Virgil C. Summer Nuclear Station South Carolina Electric & Gas

This report is being submitted as a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released from the Virgil C. Summer Nuclear Station. This report satisfies the requirements in Sections 6.9.1.8 and 6.14.2 of Technical Specifications, Section 1.6.2 of the Offsite Dose Calculation Manual (ODCM) and 10CFR50.36(a). Also included is an assessment of radiation doses from plant releases.

A brief discussion of the Supplemental Information and Tables 2 through 6 is presented in Sections A through D. An evaluation of the radiological impact on man due to operation of the Virgil C. Summer Nuclear Station is presented in Section E and Table 1. Section F provides information on one reportable incident occurring during the period. No changes were made to the Offsite Dose Calculation Manual during the period.

A. Supplemental Information

Regulatory limits for doses and maximum permissible concentrations presented in Supplemental Information are from the Virgil C. Summer Nuclear Station Technical Specifications and 40 CFR 190. Average energy (E) is not applicable to the method for determining release rate limits for fission and activation gaseous effluents; therefore, it has been omitted.

B. Gaseous Effluents

Gaseous effluents released from ground level are summarized in Tables 2 and 3. An elevated release pathway does not exist at Virgil C. Summer Nuclear Station. The errors for gaseous effluent totals are given as the square root of the sum of squares of counting errors and flow or volume measurement errors. A systematic error of 15% has been added to estimate total error. Cumulative doses are discussed in Section E.

C. Liquid Effluents

Liquid effluents are summarized in Tables 4 and 5. Estimated total errors are expressed as in Section B above.

D. Solid Waste Shipments

Solid waste shipments are summarized in Table 6. Curie content of radioactive waste packages is determined by dose rates and/or analysis of samples by gamma spectroscopy. The total error for each type of Curie content determination is conservatively estimated to be the sum of a 15% systematic error and a 20% photon response error for the detector used.

E. Radiological Impact on Man

Potential doses to the maximum exposed individual in the unrestricted area were calculated using measured plant gaseous effluent and meteorological data in accordance with the ODCM. The source term included three (3) waste gas decay tank (WGDT) batch releases, 5.88 days of 6-inch and 8.39 days of 36-inch Reactor Building purge releases and a continuous six month main plant vent release. Doses are summarized in Table 1. The total activities released are

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

*Virgil C. Summer Nuclear Station
South Carolina Electric & Gas*

presented in Tables 2 and 3. Air doses to the maximum exposed individual due to noble gases were $1.02\text{E-}3$ and $2.18\text{E-}3$ mrad for gamma and beta, respectively. The maximum organ dose attributed to the releases was $6.49\text{E-}3$ mrem for the six month period.

Measured plant liquid effluent data was used to calculate estimates of doses to individuals in accordance with the Offsite Dose Calculation Manual. The source term consisted of the isotopic contents of 146 Waste Monitor Tank batch releases, 8.61 days of Steam Generator Blowdown release and a continuous Turbine Building Sump release. Doses are summarized in Table 1 and total radioactivity released is described in Tables 4 and 5. The total body dose to the maximally exposed individual due to the release of radioactive liquid was $5.56\text{E-}3$ mrem. The maximum organ dose was $6.84\text{E-}3$ mrem to the gastrointestinal tract, lower large intestine (GILLI) during the six month period.

Table 1

GASEOUS AND LIQUID EFFLUENT DOSE SUMMARY

ODCM Section	Gaseous Limits	First Quarter, 1994		Second Quarter, 1994	
		Dose	Percent of Limit	Dose	Percent of Limit
1.2.3.1a,b	5 mrad gamma/qtr. 10 mrad gamma/yr.	$1.01\text{E-}3$ mrad	$2.03\text{E-}2$ $1.01\text{E-}2$	$5.19\text{E-}6$ mrad	$1.04\text{E-}4$ $1.02\text{E-}2^*$
(January-June total gamma air dose: $1.02\text{E-}3$ mrad)					
1.2.3.1a,b	10 mrad beta/qtr. 20 mrad beta/yr.	$1.91\text{E-}3$ mrad	$1.91\text{E-}2$ $9.55\text{E-}3$	$2.70\text{E-}4$ mrad	$2.70\text{E-}3$ $1.09\text{E-}2^*$
(January-June total beta air dose: $2.18\text{E-}3$ mrad)					
1.2.4.1a,b	7.5 mrem/organ/qtr. 15 mrem/organ/yr.	$6.44\text{E-}3$ mrem	$8.59\text{E-}2$ $4.29\text{E-}2$	$5.30\text{E-}5$ mrem	$7.06\text{E-}4$ $4.33\text{E-}2^*$
(January-June maximum exposed organ dose: $6.49\text{E-}3$ mrem)					
<u>Liquid Limits</u>					
1.1.3.1a,b	1.5 mrem/qtr. 3.0 mrem/yr.	$2.40\text{E-}3$ mrem	$1.60\text{E-}1$ $8.01\text{E-}2$	$3.16\text{E-}3$ mrem	$2.11\text{E-}1$ $1.85\text{E-}1^*$
(January-June whole body dose: $5.56\text{E-}3$ mrem)					
1.1.3.1a,b	5 mrem/organ/qtr. 10 mrem/organ/yr.	$3.66\text{E-}3$ mrem†	$7.32\text{E-}2$ $3.66\text{E-}2$	$3.20\text{E-}3$ mrem†	$6.40\text{E-}2$ $6.84\text{E-}2^*$
(January-June maximum exposed organ dose: $6.84\text{E-}3$ mrem†)					

* Includes contribution from previous quarters.

† Maximum organ dose for quarters 1 and 2 were to the gastrointestinal tract, lower large intestine(GILLI) and the liver, respectively. Max. for the six month period was to the GILLI.

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

*Virgil C. Summer Nuclear Station
South Carolina Electric & Gas*

Dose rates and concentrations were below the limits specified in Supplemental Information, Section 2a, b and c during all the effluent releases.

Radiation doses from radioactive effluents to workers at the Fairfield Hydro Station were calculated for the six-month period to be $4.80\text{E-}5$ and $1.03\text{E-}4$ mrad for gamma and beta, respectively.

Radiation doses from nearby uranium fuel cycle sources were not assessed. ODCM, Sections 1.3.1 and B/1.3 establish a five (5) mile limit beyond which doses from nearby plants are insignificant. There are no uranium fuel cycle plants within a five (5) mile radius of Virgil C. Summer Nuclear Station.

F. Offsite Dose Calculation Manual (ODCM) Reportable Incident

This report is made in accordance with Section 1.2.1.1.b of the ODCM for a failure to perform a quarterly surveillance of the Reactor Building Purge System Radiation Monitor, RM-A4. The verification of the low flow alarm (Table 1.2-2, item 3.d) was inadvertently deleted from the surveillance procedure in January of 1983 when the procedure was revised by the maintenance organization. During the period of noncompliance, the operability of the alarm function was still verified during the 18 month calibration of the monitor. The procedure deficiency was corrected on January 20, 1994 and plant personnel confirmed that all other required surveillances were being performed in accordance with the ODCM.

G. Offsite Dose Calculation Manual

The Virgil C. Summer Nuclear Station (VCSNS) Offsite Dose Calculation Manual was not revised during the affected six month period.

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

*Virgil C. Summer Nuclear Station
South Carolina Electric & Gas*

Supplemental Information

1. Regulatory Limits:

a. Fission and Activation Gases:

The air dose to an individual due to noble gases released in gaseous effluents shall be limited to less than or equal to 5 mrad for gamma radiation and 10 mrad for beta radiation during any calendar quarter and 10 mrad for gamma radiation and 20 mrad for beta radiation during any calendar year (ODCM, Section 1.2.3.1).

b. Iodines, Particulates (half-lives > 8 days) and Tritium:

The dose to an individual from radioiodines, tritium and radioactive materials in particulate form with half-lives greater than 8 days in gaseous effluents shall be limited to less than or equal to 7.5 mrem to any organ during any calendar quarter and 15 mrem to any organ during any calendar year (ODCM, Section 1.2.4.1).

c. Liquid Effluents:

The dose or dose commitment to an individual from radioactive materials in liquid effluents released shall be limited to less than or equal to 1.5 mrem to the total body and 5 mrem to any organ during any calendar quarter and 3 mrem to the total body and 10 mrem to any organ during any calendar year (ODCM, Section 1.1.3.1).

d. All Sources:

The annual dose equivalent shall not exceed 25 mrem to the whole body, 75 mrem to the thyroid and 25 mrem to any other organ (40 CFR 190).

2. Maximum Permissible Concentrations:

a. Fission and Activation Gases:

The dose rate in unrestricted areas due to radioactive materials released in gaseous effluents shall be limited to less than or equal to 500 mrem/year to the total body and less than or equal to 3000 mrem/year to the skin (ODCM, Section 1.2.2.1).

b. Iodines, Particulates (half-lives > 8 days) and Tritium:

The dose rate in unrestricted areas due to radioactive materials in effluents shall be limited to less than or equal to 1500 mrem/year to any organ (ODCM, Section 1.2.2.1).

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

*Virgil C. Summer Nuclear Station
South Carolina Electric & Gas*

Supplemental Information

c. Liquid Effluents:

The concentration of radioactive materials released from the site shall be limited to the concentrations specified in 10 CFR 20, Appendix B, Table II, Column 2 for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to $2\text{E-}4$ $\mu\text{Ci/ml}$ total activity (ODCM, Section 1.1.2.1).

3. Average Energy:

Not Applicable

4. Measurements and Approximations of Total Radioactivity:

- a. Fission and activation gases: Gamma spectrometry [Ge(Li) or HPGe]
- b. Iodines: Gamma spectrometry [Ge(Li) or HPGe]
- c. Particulates: Gamma spectrometry [Ge(Li) or HPGe], beta proportional counting, alpha proportional counting
- d. Tritium: Liquid scintillation
- e. Liquid effluents: Gamma spectrometry [Ge(Li) or HPGe], liquid scintillation (H-3), beta proportional counting, alpha proportional counting

5. Batch Releases:

a. Gaseous:

- 1. Number of batch releases: 3
- 2. Total time period for batch releases: $7.70\text{E}+2$ min.
- 3. Maximum time period for a batch release: $4.70\text{E}+2$ min.
- 4. Average time period for a batch release: $2.57\text{E}+2$ min.
- 5. Minimum time period for a batch release: $1.40\text{E}+2$ min.

b. Liquid:

- 1. Number of batch releases:
66 for first quarter, 1994
80 for second quarter, 1994

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Virgil C. Summer Nuclear Station
South Carolina Electric & Gas

Supplemental Information

2. Total time period for batch releases:
5.19E + 3 min. for first quarter, 1994
5.79E + 3 min. for second quarter, 1994
 3. Maximum time period for a batch release:
1.64E + 2 min. for first quarter, 1994
8.30E + 1 min. for second quarter, 1994
 4. Average time period for batch releases:
7.86E + 1 min. for first quarter, 1994
7.24E + 1 min. for second quarter, 1994
 5. Minimum time period for a batch release:
5.70E + 1 min. for first quarter, 1994
6.20E + 1 min. for second quarter, 1994
 6. Average stream flow during periods of release of effluent into a flowing stream:
6.03E + 6 gpm for first quarter, 1994
4.78E + 6 gpm for second quarter, 1994
6. Abnormal Releases:
- a. Gaseous:
 1. Number of releases: 0
 2. Total activity released: 0
 - b. Liquid:
 1. Number of releases: 0
 2. Total activity released: 0

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Table 2

GASEOUS EFFLUENTS-SUMMATION OF ALL RELEASES

		First Quarter	Second Quarter	Est.Total Error, %
A. Fission & activation gases				
1. Total release	Ci	7.50E+0	7.11E-1	3.13E+1
2. Average release rate for period	μCi/sec	9.65E-1	9.05E-2	
3. Percent of technical specification limit	%	*	*	
B. Iodines				
1. Total iodine-131	Ci	0	2.78E-6	5.35E+1
2. Average release rate for period	μCi/sec	0	3.54E-7	
3. Percent of technical specification limit	%	**	**	
C. Particulates				
1. Particulates with half-lives > 8 days	Ci	0	0	N/A
2. Average release rate for period	μCi/sec	0	0	
3. Percent of technical specification limit	%	**	**	
4. Gross alpha radioactivity	Ci	0	0	
D. Tritium				
1. Total release	Ci	1.22E+1	8.36E-5	1.87E+1
2. Average release rate for period	μCi/sec	1.57E+0	1.06E-5	
3. Percent of technical specification limit	%	**	**	

* Calculated as a percent of dose limits found in Supplemental Information, Section 1a. First quarter values were 2.03E-2% and 1.01E-2% of the quarterly and cumulative annual gamma dose limits, respectively and 1.91E-2% and 9.55E-3% of the quarterly and cumulative annual beta dose limits, respectively. Second quarter values were 1.04E-4% and 1.02E-2%† of the quarterly and cumulative annual gamma dose limits, respectively, and 2.70E-3% and 1.09E-2%† of the quarterly and cumulative annual beta dose limits, respectively.

** Calculated as a percent of dose limits found in Supplemental Information, Section 1b. The sum of these values for the first quarter was 8.59E-2% and 4.29E-2% of the quarterly and cumulative annual organ dose limits, respectively. The sum of these values for the second quarter was 7.06E-4% and 4.33E-2%† of the quarterly and cumulative annual organ dose limits, respectively.

† Note: Second quarter values are the sum of the first and second quarter values compared to annual dose limits.

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT
January - June, 1994

Table 3

GASEOUS EFFLUENTS-GROUND-LEVEL RELEASES

Nuclides Released	Unit	Continuous Mode		Batch Mode	
		First Quarter	Second Quarter	First Quarter	Second Quarter
1. Fission gases					
Krypton-85	Ci	0	0	0	6.69E-1
Krypton-85m	Ci	0	0	0	0
Krypton-87	Ci	0	0	0	0
Krypton-88	Ci	0	0	0	0
Xenon-133	Ci	6.35E+0	1.21E-5	0	3.62E-2
Xenon-135	Ci	1.07E+0	0	0	0
Xenon-135m	Ci	0	0	0	0
Xenon-138	Ci	0	0	0	0
Others: Ar-41	Ci	8.52E-2	8.55E-5	0	0
Xe-131m	Ci	0	0	0	5.99E-3
Unidentified	Ci	0	0	0	0
Total for period	Ci	7.50E+0	9.76E-5	0	7.11E-1

2. Iodines

Iodine-131	Ci	0	2.78E-6	0	0
Iodine-132	Ci	0	0	0	0
Iodine-133	Ci	0	2.56E-6	0	0
Iodine-134	Ci	0	0	0	0
Iodine-135	Ci	0	0	0	0
Total for period	Ci	0	5.34E-6	0	0

3. Particulates

Strontium-89	Ci	0	0	0	0
Strontium-90	Ci	0	0	0	0
Cesium-134	Ci	0	0	0	0
Cesium-137	Ci	0	0	0	0
Barium-Lanth.-140	Ci	0	0	0	0
Others: none	Ci	0	0	0	0
Unidentified	Ci	0	0	0	0
Total for period	Ci	0	0	0	0

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Table 4

LIQUID EFFLUENTS - SUMMATION OF ALL RELEASES

	Unit	First Quarter	Second Quarter	Est.Total Error, %
A. Fission & activation products				
1. Total release (not including tritium,gases, alpha)	Ci	2.59E-2	2.41E-2	2.15E+1
2. Average diluted concentration during period	μCi/ml	7.05E-11	6.74E-11	
3. Percent of applicable limit	%	*	*	
B. Tritium				
1. Total release	Ci	2.04E+2	2.62E+2	1.83E+1
2. Average diluted concentration during period	μCi/ml	5.54E-7	7.31E-7	
3. Percent of applicable limit	%	*	*	
C. Dissolved and entrained gases				
1. Total release	Ci	1.67E-2	2.13E-2	3.87E+1
2. Average diluted concentration during period	μCi/ml	4.53E-11	5.96E-11	
3. Percent of applicable limit	%	*	*	
D. Gross alpha radioactivity				
1. Total release	Ci	0	0	N/A
E. Volume of waste released (prior to dilution)				
	liters	3.00E+7	1.79E+7	3.00E+0
F. Volume of dilution water used during period				
	liters	3.67E+11	3.58E+11	4.30E+0

* See following page.

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Table 4

LIQUID EFFLUENTS-SUMMATION OF ALL RELEASES

- * Calculated as a percent of dose limits found in Supplemental Information, Section 1c. The sum of these values for the first quarter was $1.60\text{E-}1\%$ and $8.01\text{E-}2\%$ of the respective quarterly and cumulative annual whole body dose limits and $7.32\text{E-}2\%$ and $3.66\text{E-}2\%$ of the respective quarterly and cumulative annual organ dose limits. The sum of these values for the second quarter was $2.11\text{E-}1\%$ and $1.85\text{E-}1\%$ † of the respective quarterly and cumulative annual whole body dose limits and $6.40\text{E-}2\%$ and $6.84\text{E-}2\%$ † of the respective quarterly and cumulative annual organ dose limits. Dose to the gastrointestinal tract, lower large intestine (GILLI) was the most limiting organ dose for the first quarter and first six (6) months of 1994. Dose to the liver was the most limiting organ dose for the second quarter of 1994.
- † Note: Second quarter values are the sum of the first and second quarter values compared to annual dose limits.

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Table 5

LIQUID EFFLUENTS

†Nuclides Released	Unit	Continuous Mode		Batch Mode	
		First Quarter	Second Quarter	First Quarter	Second Quarter
Strontium-89	Ci	0	0	0	0
Strontium-90	Ci	0	0	0	0
Cesium-134	Ci	0	0	1.37E-5	5.44E-5
Cesium-137	Ci	0	0	7.08E-5	1.54E-4
Iodine-131	Ci	0	6.73E-6	1.20E-6	2.65E-5
Cobalt-58	Ci	0	0	3.61E-3	1.52E-2
Cobalt-60	Ci	8.59E-5	0	1.37E-2	2.78E-3
Iron-59	Ci	0	0	9.33E-5	1.60E-5
Zinc-65	Ci	0	0	5.69E-5	0
Manganese-54	Ci	0	0	2.22E-3	6.96E-4
Chromium-51	Ci	0	0	3.78E-4	5.32E-6
Zirconium-Niobium-95	Ci	0	0	9.36E-4	1.66E-5
Molybdenum-99	Ci	0	0	0	0
Technetium-99m	Ci	0	0	0	0
Barium-Lanth.-140	Ci	0	0	4.26E-5	8.29E-6
Cerium-141	Ci	0	0	0	0
Other: Na-24	Ci	2.22E-5	2.69E-5	0	5.40E-6
Fe-55	Ci	0	0	3.54E-3	3.08E-3
Co-57	Ci	0	0	5.89E-5	2.72E-5
Ag-110m	Ci	0	0	2.90E-5	0
Sn-113	Ci	0	0	6.25E-6	0
Sb-124	Ci	0	0	4.59E-6	2.48E-4
Sb-125	Ci	0	0	7.36E-4	1.59E-3
I-132	Ci	6.94E-5	3.62E-5	0	1.66E-6
I-133	Ci	6.96E-5	3.73E-5	0	0
I-134	Ci	4.77E-5	3.51E-5	0	2.02E-6
I-135	Ci	9.67E-5	5.16E-5	0	0
Cs-138	Ci	3.80E-5	0	0	1.61E-5
Unidentified	Ci	0	0	0	0
Total for period (above)	Ci	4.30E-4	1.94E-4	2.55E-2	2.39E-2

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1994

Table 5 (continued)

LIQUID EFFLUENTS

†Nuclides Released	Unit	Continuous Mode		Batch Mode	
		First Quarter	Second Quarter	First Quarter	Second Quarter
Xenon-133	Ci	0	0	1.19E-2	1.99E-2
Xenon-135	Ci	0	0	6.65E-6	5.20E-6
Other: Kr-85	Ci	0	0	4.07E-3	1.19E-3
Xe-131m	Ci	0	0	5.93E-4	1.18E-4
Xe-133m	Ci	0	0	0	9.89E-5
Xe-135m	Ci	5.63E-5	0	0	0
Total entrained gases	Ci	5.63E-5	0	1.66E-2	2.13E-2

† Tritium not included. See Table 4 for tritium numbers.

SEMIANNUAL EFFLUENT AND WASTE DISPOSAL REPORT

January - June, 1992

Table 6

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

A. Solid Waste Shipped Offsite for Burial or Disposal (Not irradiated fuel)

1. Type of waste	Unit	6-month Period	Est. Total Error, %
a. Spent resins, filter sludges, evaporator bottoms, etc.	m ³ Ci	1.76E + 1 1.10E + 0	3.50E + 1
b. Dry compressible waste, contaminated equip., etc.	m ³ Ci	1.53E + 1 7.50E - 1	3.50E + 1
c. Irradiated components, control rods, etc.	m ³ Ci	0 0	N/A
d. Other (describe)	m ³ Ci	0 0	N/A

2. Estimate of major nuclide composition (by type of waste)

a. Fe-55	%	3.06E + 1
Co-60	%	2.50E + 1
Co-58	%	1.53E + 1
Ni-63	%	1.25E + 1
Mn-54	%	7.14E + 0
Cs-137	%	2.25E + 0
Sb-125	%	1.14E + 0
b. Fe-55	%	5.08E + 1
Co-60	%	2.31E + 1
Ni-63	%	1.04E + 1
Cs-137	%	6.10E + 0
Mn-54	%	3.50E + 0
Cs-134	%	2.20E + 0
c. None	%	N/A
d. None	%	N/A

*All nuclides with concentrations above 1.0% are listed in descending order by activity level.

3. Solid Waste Disposition (6 month period)

Number of Shipments	Mode of Transportation	Destination
43†	Truck	Barnwell, SC

†Note: 42 of these are partial shipments of DAW from waste processor to Barnwell, SC.

B. Irradiated Fuel Shipments (Disposition)

Number of Shipments	Mode of Transportation	Destination
0	N/A	N/A