

CONTROL BLOCK										(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)																											
01	N	C	M	G	S	1	2	0	0	-	0	0	0	0	0	-	0	0	3	4	1	1	1	1	4	5											
LICENSEE CODE					LICENSEE NUMBER										LICENSEE TYPE					CAT 15																	
CONT																																					
01	L	5	0	5	0	0	0	3	6	9	7	0	6	0	2	8	3	8	0	7	0	6	8	3	9												
REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE							
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)																																					
While in Mode 3, during a reactor trip transient (reactor had tripped due to																																					
steam generator 1B Lo-Lo level), main steam code safety relief valve 1SV-9 lifted																																					
while the main steam pressure (app. 1121 psig) was below the acceptable lift																																					
setpoint range (1179 to 1202 psig). The valve was declared inoperable per T.S.																																					
3.7.1.1 which is reportable per T.S.6.9.1.11(b) and similar to RO-370/83-12.																																					
There were still 4 operable code safety valves on S/G 1C, and 1SV-9 was still																																					
capable of relieving steam pressure. Health and safety of the public were																																					
unaffected.																																					
SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE																																					
09	C	C	11	E	12	B	13	V	A	L	V	E	X	14	J	15	B	16																			
LEADER REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.																																					
17	8	3		0	3	7		0	3		L		0																								
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NRC-4 FORM SUB PRIME COMP. SUPPLIER COMPONENT MANUFACTURER (26)																																					
18	Z	19	Z	20	Z	21	0	0	0	0	N	23	N	24	L	25	C	7	1	0																	
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)																																					
Apparently the valve's (Crosby, nozzle type C safety valve, style HA-65W) set-																																					
point drifted from its original setting of 1200 psig. The adjusting nut is held																																					
stationary with a cotter pin, however minor drift (possibly caused by vibration																																					
and heat) is not uncommon. All power range high flux setpoints were adjusted																																					
per T.S.3.7.1.1 action a, and steam pressure was kept well below the lift set																																					
point until the setpoint was adjusted to its correct value.																																					
FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)																																					
15	X	28	0	0	0	29	Mode 3	A	31	Control Room Alarms/Indications																											
ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)																																					
16	Z	33	Z	34	N/A																																
PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)																																					
17	0	0	0	37	Z	38	N/A																														
PERSONNEL INJURIES NUMBER DESCRIPTION (41)																																					
18	0	0	0	40			N/A																														
LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)																																					
19	Z	42					N/A																														
PUBLICITY ISSUED DESCRIPTION (45)																																					
20	N	44					N/A																														
NAME OF PREPARER Phillip B. Nardoci PHONE (704) 373-7432																																					

USNRC REGION II
DUKE POWER COMPANY
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CHARLOTTE, N.C. 28242

HAL B. TUCKER
VICE PRESIDENT
NUCLEAR PRODUCTION

83 JUL 12 A9:23

TELEPHONE
(704) 373-4531

July 6, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street NW, Suite 2900
Atlanta, Georgia 30303

Re: McGuire Nuclear Station Unit 1
Docket No. 50-369

Dear Mr. O'Reilly:

Please find attached Reportable Occurrence Report RO-369/83-37. This report concerns T.S. 3.7.1.1, "All main steam line code safety valves associated with each steam generator shall be operable with lift settings as specified in Table 3.7-3". This incident was considered to be of no significance with respect to the health and safety of the public.

Very truly yours,

H.B. Tucker 1/5/83
Hal B. Tucker

PBN:jfw
Attachment

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Records Center
Institute of Nuclear Power Operations
1100 Circle 75 Parkway, Suite 1500
Atlanta, Georgia 30339

Mr. W. T. Orders
NRC Resident Inspector
McGuire Nuclear Station

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