

LICENSEE EVENT REPORT

CONTROL BLOCK:

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 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0	1	N	C	B	E	P	2	2	0	0	-	0	0	0	0	0	0	0	0	3	4	1	1	1	1	4			5	
7	8	LICENSEE CODE						14	15	LICENSE NUMBER										25	26	LICENSE TYPE					30	57	CAT	58

CON'T

0 1 7 8
REPORT SOURCE L 6 0 5 0 - 0 3 2 4 7 0 6 0 5 8 3 8 0 6 0 5 8 3 9
60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | During unit power operation, while withdrawing control rods to increase reactor

0 3 | power level it was determined that control rod 26-19 did not have RTGB position

0 4 | indication at positions "02," "03," "06," "07," and "09." This event did not

0 5 | affect the health and safety of the public.

0 6 |

0 7 |

Technical Specifications 3.1.3.7, 6.9.1.9b

0 9		SYSTEM CODE I D		CAUSE CODE X		CAUSE SUBCODE X		COMPONENT CODE Z Z Z Z Z Z						COMP. SUBCODE Z		VALVE SUBCODE Z	
7	8	9	10	11	12	12	13	13	14	15	16	17	18	19	20		
17 LER/RO REPORT NUMBER		EVENT YEAR 8 3		SEQUENTIAL REPORT NO. 0 5 6		OCCURRENCE CODE 0 3		REPORT TYPE L		REVISION NO. 0							
21	22	23	24	25	26	27	28	29	30	31	32						
ACTION TAKEN X		FUTURE ACTION X		EFFECT ON PLANT Z		SHUTDOWN METHOD Z		HOURS 0 0 0 0		ATTACHMENT SUBMITTED N		NPRD-4 FORM SUB Y		PRIME COMP. SUPPLIER N		COMPONENT MANUFACTURER G 0 8 0	
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Initial troubleshooting of this event has determined the most likely cause is a

1 1 problem with the rod position indication probe. During the planned Fall, 1983

1 2 unit outage this problem will be fully investigated and resolved. As this problem

1 3 does not constitute a safety hazard no further corrective action regarding this event

1 4 is planned.

FACILITY STATUS (28) F 5 1
 % POWER 0 4 8 (29)
 OTHER STATUS (30) NA
 METHOD OF DISCOVERY (31) A Operational Event
 DISCOVERY DESCRIPTION (32)
 ACTIVITY CONTENT
 RELEASED OF RELEASE (33) Z 6 1
 AMOUNT OF ACTIVITY (35) NA
 LOCATION OF RELEASE (36)
 PERSONNEL EXPOSURES
 NUMBER (37) 0 0 0 7 1
 TYPE (38) Z
 DESCRIPTION (39)
 PERSONNEL INJURIES
 NUMBER (40) 0 0 0 8 1
 DESCRIPTION (41)
 LOSS OF OR DAMAGE TO FACILITY
 TYPE (42) Z 9 1
 DESCRIPTION (43)
 PUBLICITY
 ISSUED (44) N 0 2
 DESCRIPTION (45)
 8307150242 830605
 PDR ADOCK 05000324
 S PDR
 NRC USE ONLY

NAME OF PREPARER M. J. Pastva, Jr.

PHONE: 919-457-9521

USNRC REGION II
ATL CP&L

83 JUL 12 09:25
Columbia Power & Light Company

Brunswick Steam Electric Plant
P. O. Box 10429
Southport, NC 28461-0429

July 5, 1983

FILE: B09-13510C
SERIAL: BSEP/83-2127

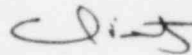
Mr. James P. O'Reilly, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street N.W.
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-324
LICENSE NO. DPR-62
LICENSEE EVENT REPORT 2-83-56

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 2, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,



C. R. Dietz, General Manager
Brunswick Steam Electric Plant

RMP/jo/LETJ03

Enclosure

cc: Mr. R. C. DeYoung
NRC Document Control Desk

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