

## LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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0 1 REPORT SOURCE L 6 0 5 0 - 0 3 2 5 7 0 6 0 6 8 3 8 0 7 0 6 8 3 9  
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## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During an ongoing unit refueling outage a routine Shift Operating Supervisory review

0 3 on 6-06-83 determined that on 6-04-83 the SLC System had been unknowingly rendered

0 4 inoperable for one hour and 20 minutes when the valve motor power supply breakers of

0 5 RWCU System inboard and outboard PCIVs, 1-G31-F001 and F004 were intentionally

0 6 deenergized while the valves were open. This event did not affect the health and

0 7 safety of the public.

0 8 Technical Specifications 3.1.5, 6.9.1.9b  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

0 9 SYSTEM CODE R B 11 CAUSE CODE A 12 CAUSE SUBCODE A 13 COMPONENT CODE C K T B R K 14 COMP. SUBCODE A 15 VALVE SUBCODE Z 16

17 LER/RO REPORT NUMBER 8 3 21 22 SEQUENTIAL REPORT NO. 0 2 5 24 26 OCCURRENCE CODE 0 3 28 29 REPORT TYPE L 30 31 REVISION NO. 0 32

ACTION TAKEN X 18 FUTURE ACTION Z 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 22 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER Z 25 COMPONENT MANUFACTURER Z 9 9 9 26

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 Responsible personnel failed to adequately identify in the technical specification

1 1 interpretation log index that an interpretation of the applicable technical specifica-

1 2 tion had been made. The log index was updated to reflect all pages in the log; and

1 3 and Control Room copies of technical specifications, for both Unit Nos. 1 and 2, were

1 4 annotated, as required, to ensure the log is utilized whenever a technical specifica-

1 5 tion interpretation is made.

1 5 FACILITY STATUS H 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Routine Supervisory Review 32

1 6 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43

2 0 PUBLICATION ISSUED N 44 DESCRIPTION NA 45

8307150102 830706  
PDR ADOCK 05000325  
S PDR

NRC USE ONLY

NAME OF PREPARER M. J. Pastva, Jr.

PHONE 919-457-9521

LER ATTACHMENT - RO #1-83-25

Facility: Unit No. 1

Event Date: June 6, 1983

During an ongoing unit refueling outage on June 4, 1983, the valve operator power supply breakers of RWCU System suction valves 1-G31-F001 and F004 were deenergized to allow the performance of maintenance on an RWCU System instrument. At the time the RWCU System was being utilized for reactor vessel level control and it was desired not to isolate the RWCU System; therefore, the decision was then made to deenergize the breakers with the valves in the open position. Prior to deenergizing the subject breakers, a review of Technical Specifications 3.1.5, 3.3.2, 3.6.3, the plant standing instructions, and the index section of the technical specifications interpretation log had been done to ensure the SLC System operability would not be affected. The review had determined no technical specifications impact with respect to deenergizing the breakers.

As a result of this event, a review of the technical specifications interpretation log index determined that it was not up to date. An evaluation had been entered in the interpretation log which identified the inconsistency between the SLC specification and the RWCU isolation specification and provided guidance; however, the log entry had not been reflected in the log index. This allowed the section of the log applicable to SLC System operability with the RWCU System subject valve breakers deenergized to be overlooked. This event was discovered on June 6, 1983, during a routine review of plant documentation by the Shift Operating Supervisor.

Following this discovery, the log index was updated to reflect all pages of the log. The technical specifications for both Unit Nos. 1 and 2 were annotated as required to ensure the technical specification interpretation log is utilized when required. Also, personnel responsible for providing interpretation log entries have been instructed to ensure that an index entry has been made prior to completing the respective log entry. As the corrective actions taken are felt to adequately correct the root cause of this event, no further action regarding this event is planned.

USNRC REGION II  
ATLANTA, GEORGIA  
**CP&L**

83 JUL 12 4:22  
Carolina Power & Light Company

Brunswick Steam Electric Plant  
P. O. Box 10429  
Southport, NC 28461-0429

July 6, 1983

FILE: B09-13510C  
SERIAL: BSEP/83-2152

Mr. James P. O'Reilly, Administrator  
U. S. Nuclear Regulatory Commission  
Region II, Suite 3100  
101 Marietta Street N.W.  
Atlanta, GA 30303

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
LICENSEE EVENT REPORT 1-83-25

Dear Mr. O'Reilly:

In accordance with Section 6.9.1.9b of the Technical Specifications for Brunswick Steam Electric Plant, Unit No. 1, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-0161, July 1977.

Very truly yours,

*C.R. Dietz*  
C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

RMP/joh/LETJH1

Enclosure

cc: Mr. R. C. DeYoung  
NRC Document Control Desk

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