WOLF CREEK
NUCLEAR OPERATING CORPORATION

Neil S. "Buzz" Carns
Chairman, President and
Chief Executive Officer

August 25, 1995

WM 94-0108

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, D. C. 20555

Reference: Letter dated May 12, 1994, from W. D. Reckley, NRC, to N. S. Carns, WCNOC
Subject: Docket No. 50-482: Reactor Vessel Structural Integrity

Gentlemen:

This letter submits corrected tables of pressurized thermal shock (PTS) and upper-shelf energy (USE) information as requested by the Reference. The Reference transmitted tables that contained PTS and USE information for Wolf Creek Generating Station and requested that an accuracy review be performed on the information contained in the tables. This information was based on data that was taken from Wolf Creek Nuclear Operating Corporation's response to Generic Letter 92-01, Revision 1, "Reactor Vessel Structural Integrity," and previously docketed information. Portions of this information were originally telecopied to Mr. W. D. Reckley on June 10, 1994, with subsequent changes made based on input from Westinghouse. Please review the attached corrected tables and update your database accordingly.

If you have any questions concerning this matter, please contact me at (316) 364-8831, extension 4000, or Mr. Richard D. Flannigan at extension 4500.

Very truly yours,

Neil S. Carns

Neil S. Carns

NSC/jra

Attachment

cc: L. J. Callan (NRC), w/a
D. N. Graves (NRC), w/a
W. D. Reckley (NRC), w/a
J. F. Ringwald (NRC), w/a

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ENCLOSURE 1

Summary File for Pressurized Thermal Shock

Plant Name	Beltline Ident.	Heat No. Ident.	ID Neut. Fluence at EOL	IRT_{ndt}	Method of Determin. IRT_{ndt}	Chemistry Factor	Method of Determin. CF	%Cu	%Ni
Wolf Creek EOL: 3/11/2025	Int. Shell R2005-1		2.5E19	-20°F	Plant Specific	26	Table	0.04	0.66
	Int. Shell R2005-2		2.5E19	-20°F	Plant Specific	26	Table	0.04	0.64
	Int. Shell R2005-3		2.5E19	-20°F	Plant Specific	31	Table	0.05	0.63
	Lower Shell R2508-1		2.5E19	0°F	Plant Specific	58	Table	0.09	0.67
	Lower Shell R2508-2		2.5E19	10°F	Plant Specific	37	Table	0.06	0.64
	Lower Shell R2508-3		2.5E19	40°F	Plant Specific	34.433	Calculated	0.07	0.62
	Int. and Lower Shell Axial Welds G2.06	90146	2.5E19	-50°F	Plant Specific	27.8	Table	0.04	0.04
	Int. to Lower Shell Circ. Weld E3.16	90146	2.5E19	-50°F	Plant Specific	41.022	Calculated	0.05	0.05

References

The copper and nickel contents and IRT_{ndt}s of the beltline materials are from Table B-1 of WCAP-13365. Phosphorus and sulfur contents of surveillance materials are from WCAP-13365.

The fluence is from Table 6-14 of WCAP-13365.

ENCLOSURE 2

Summary File for Upper Shelf Energy

Plant Name	Beltline Ident.	Heat No.	Material Type	1/4T USE at EOL	1/4T Neutron Fluence at EOL	Unirrad. USE	Method of Determin. Unirrad. USE
Wolf Creek EOL: 3/11/2025	Int. Shell R2005-1		A 5338-1	102	1.36E19	127	Direct
	Int. Shell R2005-2		A 5338-1	102	1.36E19	127	Direct
	Int. Shell R2005-3		A 5338-1	108	1.36E19	135	Direct
	Lower Shell R2508-1		A 5338-1	70	1.36E19	87	Direct
	Lower Shell R2508-2		A 5338-1	80	1.36E19	100	Direct
	Lower Shell R2508-3		A 5338-1	87	1.36E19	89	Direct
	Int. and Lower Shell Axial Welds G2.06	B4 90146	Linde 0091 SAW	120	1.36E19	149	Direct
	Int. to Lower Shell Circ. Weld E3.16	B4 90146	Linde 124 SAW	78	1.36E19	98	Surv. Weld

References

The copper and nickel contents and IRT_{ndt}s of the beltline materials are from Table B-1 of WCAP-13365. Phosphorus and sulfur contents of surveillance materials are from WCAP-13365.

The fluence is from Table 6-14 of WCAP-13365.

Material type for plates and USE data are from Table A-3 of WCAP-11553; material types for welds are from Table A-5 of WCAP-10015.