

Docket No. 50-336

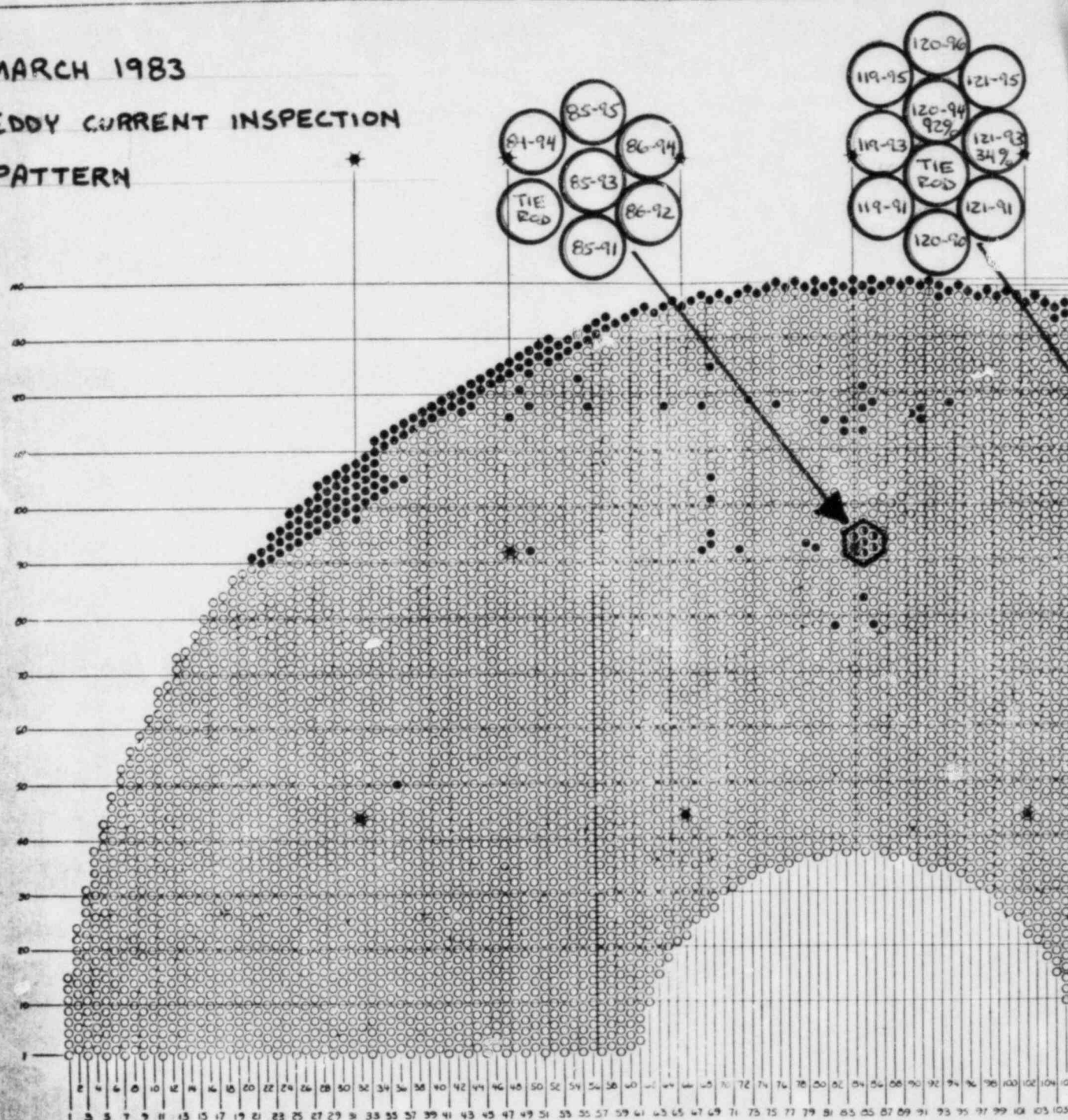
Attachment 1

Millstone Nuclear Power Station, Unit No. 2  
Proposed Revisions to Technical Specifications  
Steam Generator Inservice Inspection

March, 1983

MARCH 1983

EDDY CURRENT INSPECTION  
PATTERN



STEAM GENERATOR IDENT **1(HL)**  
TUBE SHEET VIEWED FROM PRIMARY SIDE

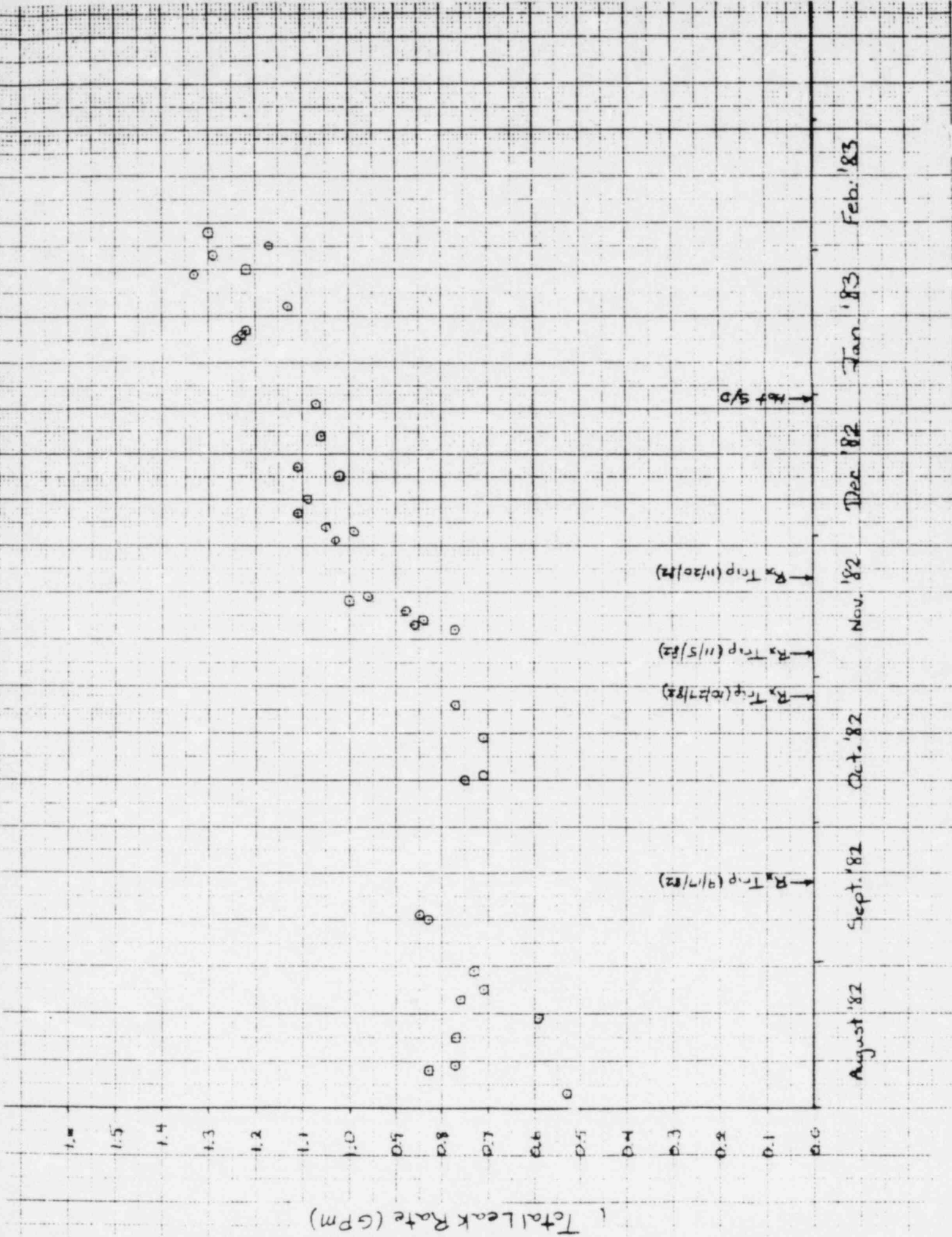


FIGURE 2

## REACTOR COOLANT SYSTEM

### SURVEILLANCE REQUIREMENTS (Continued)

4.4.5.1.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
- b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-6 at 40 month intervals fall into Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.5.1.3.a; the interval may then be extended to a maximum of once per 40 months.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-6 during the shutdown subsequent to any of the following conditions:
  1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.6.2.\*
  2. A seismic occurrence greater than the Operating Basis Earthquake.
  3. A loss-of-coolant accident requiring actuation of the engineered safeguards:
  4. A main steam line or feedwater line break.

\* For the outage commencing on March 1, 1983 only, the inservice inspection requirements of Category C-1 of Table 4.4-6 are deferred to the 1983 refueling outage.