

## U.S. NUCLEAR REGULATORY COMMISSION

## LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)  
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/5/2/1/8/3/ (8) /0/6/1/5/8/3/ (9)  
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On May 21, 1983, during Mode 1 operation, containment temperature rose to greater/  
/0/3/ / than 105 degrees F for 30 minutes. Since containment temperature was decreased /  
/0/4/ / below 105 degrees F within 8 hours as required by T.S. 3.6.1.5, the health and /  
/0/5/ / safety of the public were not affected. This event is reportable pursuant to /  
/0/6/ / T.S. 6.9.1.9.b. /  
/0/7/ / /  
/0/8/ / /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/ /S/B/ (11)	/E/ (12)	/B/ (13)	/X/X/X/X/X/X/ (14)	/Z/ (15)	/Z/ (16)
LER/RO REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.

(17) /8/3/ /-/ /0/3/2/ / / /0/3/ /L/ /-/ /0/  
ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT  
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER  
/B/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /N/ (24) /A/ (25) /C/1/4/7/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Chilled cooling water to the containment cooling coils can be supplied by two /  
/1/1/ / sources, a mechanical chiller and a steam jet chiller. The mechanical chiller /  
/1/2/ / which was in service malfunctioned. The steam jet chiller was aligned to the /  
/1/3/ / containment cooling coils. The mechanical chiller was subsequently repaired and /  
/1/4/ / returned to service. No further Corrective Action is required. /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28)	/1/0/0/ (29)	/ NA / (30)	/A/ (31)	/ Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/ /0/0/0/ (40)	/ NA /

LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/ /Z/ (42)	/ NA /

PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
/2/0/ /N/ (44)	/ NA /	/ / / / / / / / / / / / /

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

8306270430 830615  
PDR ADOCK 05000338  
S PDR

# Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

June 15, 1983

83 JUN 20 AIO: 01  
USNRC REGION II  
ATLANTA, GEORGIA

Mr. James P. O'Reilly, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, Georgia 30303

Serial No. N-83-078  
NO/CLF: 11  
Docket No. 50-338  
License No. NPF-4

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No.


Applicable Technical Specifications

LER 83-032/03L-0

T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,

  
E. Wayne Harrell  
Station Manager

for

Enclosures (3 copies)

cc: Document Control Desk (1 copy)  
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U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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