

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/6/0/6/8/3/ (8) /0/6/1/5/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On June 6, 1983, following a reactor trip from 100% power, the I-131 Dose Equiv- /
/0/3/ / alent exceeded 1.0 microcuries/gram. The first sample to exceed the limit was /
/0/4/ / taken 2.5 hours after the trip. Samples were taken at least every four hours in /
/0/5/ / accordance with Item 4a of Technical Specification Table 4.4-4. The level /
/0/6/ / returned to less than the limit within 9 hours; therefore, the public health and /
/0/7/ / safety were not affected. This event is reportable pursuant to T.S. 6.9.1.9.b /
/0/8/ / and T.S. 6.9.2. /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/ /R/C/ (11)	/X/ (12)	/Z/ (13)	/Z/Z/Z/Z/Z/Z/ (14)	/Z/ (15)	/Z/ (16)
LER/RO REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.

(17) /8/3/ /-/ /0/4/1/ / / /0/3/ /L/ /-/ /0/
ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER
/G/ (18) /Z/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /Y/ (23) /N/ (24) /N/ (25) /W/1/2/0/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / This event was caused by a fuel element defect, though not specifically identi- /
/1/1/ / fied, in the reactor core. Post trip conditions in the core enhanced the release /
/1/2/ / of fission fragments to the Reactor Coolant System which caused the iodine spike. /
/1/3/ / The accelerated sampling frequency of T.S. 3.4.8 was implemented until RCS speci- /
/1/4/ / fic activity returned to less than the limit of T.S. 3.4.8.a. /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /X/ (28)	/0/0/0/ (29)	/ NA / (30)	/C/ (31)	/ Chemistry Sample /
ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)	
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /	
PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)		
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /		
PERSONNEL INJURIES NUMBER	DESCRIPTION (41)			
/1/8/ /0/0/0/ (40)	/ NA /			
LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)			
/1/9/ /Z/ (42)	/ NA /			

PUBLICITY
ISSUED DESCRIPTION (45)
/2/0/ /N/ (44) / NA /

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

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PDR ADDCK 05000338
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NRC USE ONLY
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Virginia Electric and Power Company
North Anna Power Station, Unit No. 1
Docket No. 50-338
Report No. LER 83-041/03L-0

Attachment: Page 1 of 2

Description of Event

On June 6, 1983, following a reactor trip from 100% power, the specific activity of the reactor coolant exceeded 1.0 micro Ci/gram. The first primary coolant sample to exceed the limit was taken approximately 2.25 hours after the reactor trip and indicated a level of 1.92 microcurie/gram. Subsequent samples were taken at least every four hours in accordance with Item 4a of T.S. Table 4.4-4.

Probable Consequences of Occurrence

Since the Dose Equivalent I-131 limit was exceeded for a short period of time, < 9 hours, and the I-131 level was monitored by sampling at least every 4 hours until the I-131 level returned to less than the T.S. 3.4.8 limit, the public health and safety were not affected.

Cause of Event

The iodine spike was caused by increased fuel outgassing from a fuel defect, though not specifically identified, after a reactor trip from 100% power.

Immediate Corrective Action

The primary coolant was sampled and analyzed at the frequency required by item 4a of Technical Specifications Table 4.4-4. The specific activity was verified to be less than 1.0 microcurie/gram within nine hours.

Scheduled Corrective Action

No further action is required.

Actions Taken to Prevent Recurrence

No further action is required.

Generic Implications

There are no generic actions associated with this event.

Supplemental Information

This event is reportable as a "Thirty-Day Written Report" pursuant to T.S. 6.9.1.9.b. In addition the supplemental information required by T.S. 6.9.2 "Special Report" and by T.S. 3.4.8 is included as follows:

1. Reactor Power History 48 hours prior to the first sample in which the limit was exceeded.

June 6, 1983 - 19 hours at 100% RTP

June 5, 1983 - 24 hours at 100% RTP

June 4, 1983 - 24 hours at 100% RTP

2. Fuel Burnup by Core Region - As of June 6, 1983:

Fuel Batch 4a2 - 27,926.5 MWD/MTU

5a - 17,896.1 MWD/MTU

6a - 3,216.2 MWD/MTU

Cycle 4 Burnup - 2,956.1 MWD/MTU

3. Normal mixed bed demineralization operation 48 hours prior to and after the event. Average flowrate of 120 gpm.
4. No de-gassing operations were performed.
5. Duration of I-131 spike

<u>DATE</u>	<u>TIME</u>	<u>DOSE EQUIVALENT I-131 (MicroCurie/gr)</u>
June 6, 1983	2124	1.44
June 6, 1983	2329	1.34
June 7, 1983	0134	1.09
June 7, 1983	0400	.88
June , 1983	0758	.61



VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

June 15, 1983

USNRC REGION 1
ATLANTA, GEORGIA
83 JUN 20 AIO: 01

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-081
NO/WFS: dus
Docket No. 50-338
License No. NPF-4

Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No.

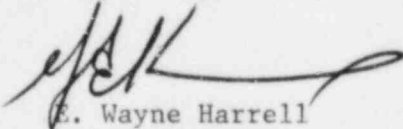
Applicable Technical Specifications

LER 83-041/03L-0

T.S. 6.9.1.9.b and T.S. 6.9.2

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,


E. Wayne Harrell
Station Manager
for

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
016 Phillips Bldg.
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

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11