

LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/2/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/9/ (7) /0/5/2/2/8/3/ (8) /0/6/1/5/8/3/ (9)
SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On May 22, 1983, with Unit 2 in Mode 4, RCS unidentified leakage exceeded the /
/0/3/ / limits specified in T.S. 4.6.2 (< 1 GPM). The source of the leak was identified /
/0/4/ / and isolated within 30 minutes. Since the leak was promptly isolated and the /
/0/5/ / Unit placed in Mode 5 within 9 hours the public health and safety were not af- /
/0/6/ / fected. This event is reportable pursuant to T.S. 6.9.1.9.b. There are no fur- /
/0/7/ / ther actions required and no generic implications to this event. /
/0/8/ /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
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/0/9/	/C/F/ (11)	/E/ (12)	/B/ (13)	/P/U/M/P/X/X/ (14)	/B/ (15)	/Z/ (16)
	LER/RO	EVENT YEAR	SEQUENTIAL	OCCURRENCE	REPORT	REVISION
	REPORT		REPORT NO.	CODE	TYPE	NO.
(17)	NUMBER	/8/3/	/-/	/0/4/2/	/-/	/0/

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
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/A/ (18) /Z/ (19) /C/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /Y/ (24) /N/ (25) /I/0/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The excessive leakage was a result of a shaft sleeve gasket failure on the "A" /
/1/1/ / Residual Heat Removal (RHR) Pump. The gasket was ripped and broken apart. No /
/1/2/ / cause for the gasket failure could be determined. The leak was stopped by secur- /
/1/3/ / ing and isolating the "A" RHR pump. Unit 2 was placed in Mode 5, the mechanical /
/1/4/ / seal and sleeve gasket replaced and the pump restored to service. /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /G/ (28)	/0/0/0/ (29)	/ NA / (30)	/A/ (31)	Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/ /0/0/0/ (40)	/ NA /

LOSS OF OR DAMAGE TO FACILITY (43) TYPE	DESCRIPTION
/1/9/ /Z/ (42)	/ NA /

PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
/2/0/ /N/ (44)	/ NA /	/ / / / / / / / / / /

NAME OF PREPARER E. Wayne Harrell PHONE (703) 894-5151

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PDR ADOCK 05000339
S PDR

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
P. O. BOX 402
MINERAL, VIRGINIA 23117

June 15, 1983

83 JUN 20 AID: 01
USNRC REGION II
ATLANTA, GEORGIA

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

Serial No. N-83-075
NO/WFS: dus
Docket No. 50-339
License No. NPF-7

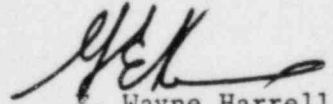
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 2.

Report No.	Applicable Technical Specifications
LER 83-042/03L-0	T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,


E. Wayne Harrell
for Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)
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