



Carolina Power & Light Company
PO Box 10429
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August 17, 1994

SERIAL: BSEP 94-0316
10 CFR 50 APPENDIX H

U. S. Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 & 50-324/LICENSE NOS. DPR-71 & DPR-62
SUBMITTAL OF REACTOR VESSEL MATERIAL SURVEILLANCE SPECIMEN TEST RESULTS
FOR BRUNSWICK UNIT 1

Gentlemen:

The purpose of this letter is to provide the NRC staff, in accordance with 10 CFR 50, Appendix H, reactor vessel material surveillance specimen test results for Carolina Power & Light Company's (CP&L) Brunswick Steam Electric Plant, Unit 1.

CP&L removed the first reactor vessel surveillance capsule for Brunswick Unit 1 on August 17, 1993. The enclosed report (Enclosure 2) summarizes the mechanical testing and fluence analysis associated with this capsule. The report includes the data required by ASTM E 185, as specified in Paragraph II.B.1 of 10 CFR 50, Appendix H, and the results of all fracture toughness tests conducted on the beltline materials in the irradiated and unirradiated conditions. As noted in the report, no changes in the Brunswick Technical Specification pressure-temperature limit curves or operating limits are required as a result of these analyses.

The attached report also updates end-of-license fluence and mechanical properties projections for the reactor vessel beltline materials based on the results of the surveillance capsule testing. By letter dated May 13, 1994, CP&L committed to evaluate the impact of the BWR Owners' Group methodology for determining initial RT_{NDT} values for beltline materials and provide the results of that assessment along with the surveillance capsule test results. This date was provided based on anticipated NRC staff approval of the BWROG methodology. To date, the revised BWROG methodology has not been finalized and submitted to the NRC staff for approval; therefore, CP&L will complete the assessment and provide the results to the NRC staff following NRC staff approval of the finalized BWROG Topical Report.

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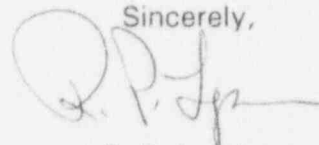
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In the May 13, 1994 letter, CP&L also committed to revise the GE NEDO reports describing the reactor vessel material surveillance programs for Brunswick Units 1 and 2. These revisions corrected the information of the reactor vessel materials that had been reversed in the earlier NEDO reports (reference Brunswick Licensee Event Report 1-94-005 and 1-94-005 Supplement 1). Enclosed for your information are copies of revised NEDO documents, NEDO-24161, Revision 1, for Brunswick Unit 1 and NEDO-24157, Revision 2, for Brunswick Unit 2 (Enclosures 3 and 4).

Please refer any questions regarding this submittal to Mr. M. A. Turkal at (910) 457-3066.

Sincerely,



R. P. Lopriore
Manager - Regulatory Affairs
Brunswick Nuclear Plant

KAH/

Enclosures

cc: Mr. S. D. Ebnetter, Regional Administrator, Region II
Mr. P. D. Milano, NRR Senior Project Manager - Brunswick Units 1 and 2
Mr. C. A. Patterson, Brunswick NRC Senior Resident Inspector
The Honorable H. Wells, Chairman - North Carolina Utilities Commission

ENCLOSURE 1

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 & DPR-62

LIST OF REGULATORY COMMITMENTS

The following table identifies those actions committed to by Carolina Power & Light Company in this document. Any other actions discussed in the submittal represent intended or planned actions by Carolina Power & Light Company. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Manager-Regulatory Affairs at the Brunswick Nuclear Plant of any questions regarding this document or any associated regulatory commitments.

Commitment	Committed date or outage
1. CP&L will evaluate the impact of the BWR Owners' Group methodology for determining initial RT_{NDT} values for beltline materials and provide the results to the NRC staff following NRC staff approval of the finalized BWROG Topical Report.	N/A

ENCLOSURE 2

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 & DPR-62

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1
REACTOR PRESSURE VESSEL SURVEILLANCE PROGRAM

SUMMARY REPORT SR-BNP1-1005-001

FIRST CAPSULE REMOVAL TEST RESULTS AND PROJECTIONS

AUGUST 1994

ENCLOSURE 3

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 & DPR-62

NEDO-24161, Rev. 1

INFORMATION ON REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM

BRUNSWICK STEAM ELECTRIC PLANT - UNIT 1

ENCLOSURE 4

BRUNSWICK STEAM ELECTRIC PLANT, UNIT 1 AND 2
NRC DOCKETS 50-325 & 50-324
OPERATING LICENSES DPR-71 & DPR-62

NEDO-24157, Rev. 2

INFORMATION ON REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM

BRUNSWICK STEAM ELECTRIC PLANT - UNIT 2