

ATTACHMENT 1

NRC DOCKET 50-366
OPERATING LICENSE NPF-5
EDWIN I. HATCH NUCLEAR PLANT UNIT 2
PROPOSAL FOR TECHNICAL SPECIFICATION CHANGE
FEEDWATER SPARGER REPAIR

The proposed change to the Technical Specifications (Appendix A to the Operating License) would be incorporated as follows:

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TABLE 3.3.2-1 (Continued)
ISOLATION ACTUATION INSTRUMENTATION

TRIP FUNCTION	VALVE GROUPS OPERATED BY SIGNAL (a)	MINIMUM NUMBER OPERABLE CHANNELS PER TRIP SYSTEM(b)(c)	APPLICABLE OPERATIONAL CONDITION	ACTION
5. <u>REACTOR CORE ISOLATION</u> <u>COOLING SYSTEM ISOLATION</u>				
a. RCIC Steam Line Flow-High (2E51-N017, 2E51-N018)	4	1	1, 2, 3	26
b. RCIC Steam Supply Pressure - Low (2E51-N019 A, B, C, D)	4, 9	2	1, 2, 3	26
c. RCIC Turbine Exhaust Diaphragm Pressure - High (2E51-N012 A, B, C, D)	4	2	1, 2, 3	26
d. Emergency Area Cooler Temperature - High (2E51-N602 A, B)	4	1	1, 2, 3	26
e. Suppression Pool Area Ambient Temperature-High (2E51-N603 A, B)	4	1	1, 2, 3	26
f. Suppression Pool Area Δ T-High (2E51-N604 A, B)	4	1	1, 2, 3	26
g. Suppression Pool Area Temperature Timer Relays (2E51-M602 A, B)	4(i)	1	1, 2, 3	26
h. Drywell Pressure - High (2E11-N011 A, B)	9	1	1, 2, 3	26
i. Logic Power Monitor (2E51-K1)	NA(h)	1	1, 2, 3	27
6. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u>				
a. Reactor Vessel Water Level-Low## (2B21-N017 A, B, C, D)	2, 5, 6, 10, 11 12	2	3, 4, 5	26
b. Reactor Steam Dome Pressure-High (2B31-N018 A, B)	11	1	1, 2, 3	28

##Not required OPERABLE during feedwater sparger bracket repair commencing on June 20, 1983.

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TABLE 3.3.2-2 (Continued)

ISOLATION ACTUATION INSTRUMENTATION SETPOINTS

<u>TRIP FUNCTION</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUE</u>
5. <u>REACTOR CORE ISOLATION</u> <u>COOLING SYSTEM ISOLATION</u>		
a. RCIC Steam Line flow - High	\leq 300% of rated flow	\leq 300% of rated flow
b. RCIC Steam Supply Pressure - Low	\geq 50 psig	\geq 50 psig
c. RCIC Turbine Exhaust Diaphragm Pressure- High	\leq 10 psig	\leq 10 psig
d. Emergency Area Cooler Temperature-High	\leq 175°F	\leq 175°F
e. Suppression Pool Area Ambient Temperature High	\leq 175°F	\leq 175°F
f. Suppression Pool Area ΔT - High	\leq 50°F**	\leq 50°F**
g. Suppression Pool Area Temperature Timer Relays	NA	NA
h. Drywell Pressure - High	\leq 2 psig	\leq 2 psig
i. Logic Power Monitor	NA	NA
6. <u>SHUTDOWN COOLING SYSTEM ISOLATION</u>		
a. Reactor Vessel Water Level - Low##	\geq 12.5 inches*	\geq 12.5 inches*
b. Reactor Steam Dome Pressure - High	\leq 135 psig	\leq 135 psig

*See Bases Figure B 3/4 3-1.

**Initial setpoint. Final setpoint to be determined during startup testing.

##Not required OPERABLE during feedwater sparger bracket repair commencing on June 20, 1983.
This involves only Group 11.

ATTACHMENT 2

NRC DOCKET 50-366
OPERATING LICENSE NPF-5
EDWIN I. HATCH NUCLEAR PLANT UNIT 2
PROPOSAL FOR TECHNICAL SPECIFICATION CHANGE
FEEDWATER SPARGER REPAIR

Pursuant to 10 CFR 170.12 (c), Georgia Power company has evaluated the attached proposed amendment to operating license NPF-5 and has determined that:

- a. The proposed amendment does not require the evaluation of a new Safety Analysis Report or rewrite of the facility license;
- b. The proposed amendment does not contain several complex issues, does not involve ACRS review, and does not require an environmental impact statement;
- c. The proposed amendment does not involve a complex issue or more than one environmental or safety issue;
- d. The proposed amendment does involve a single safety issue, namely, removing the requirement to have shutdown cooling system isolation at a level of 12.5 inches in the reactor vessel during the feedwater sparger repair.
- e. The proposed amendment is therefore a Class III amendment.