

LICENSEE EVENT REPORT

Updated Report

Previous report date 2/2/83

CONTROL BLOCK:

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 3 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO CAT 56

CONT

0 1 REPORT SOURCE L 6 0 5 0 0 0 2 4 9 7 0 1 0 5 8 3 5 0 6 0 3 8 3 9
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 During normal unit operation, after starting Rx water cleanup system, a pin-hole
0 3 leak was found on the connecting pipe weld between 3-1201-10 inch A and 3-1292-2
0 4 inch A line. The line is located outside of primary containment and all the
0 5 leakage was contained in Rx building floor drain system. The health and safety
0 6 of the public was not endangered. A similar event was reported by R.O. 82-58/03L-0
0 7 on Docket 050-237.

0 8
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

0 9 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP SUBCODE VALVE SUBCODE
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
C G 11 E 12 C 13 P I P E X X 14 C 15 Z 16
17 LER/RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
8 3 0 0 3 0 3 X 1
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER
B 18 A 19 Z 20 Z 21 0 0 0 0 N 23 Y 24 N 25 G 0 8 0 26
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of the pinhole leak is suspected to be IGSCC (Intergranular stress
1 1 corrosion cracking). The crack was ground out and rewelded. During the
1 2 upcoming Unit 3 refueling outage, a permanent repair will be implemented. If
1 3 the exact cause is determined at that time, a supplement will be submitted.

1 4
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

1 5 FACILITY STATUS % POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
E 28 0 9 9 29 N/A A 31 Operator Observation

1 6 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
Z 33 Z 34 N/A N/A

1 7 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
0 0 0 37 Z 38 N/A

1 8 PERSONNEL INJURIES NUMBER DESCRIPTION (41)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
0 0 0 40 N/A

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
Z 42 N/A

2 0 PUBLICITY ISSUED DESCRIPTION (45)
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50
N 44 N/A

NAME OF PREPARER S. Merritt

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8306270111 830603
PDR ADOCK 05000249
S PDR

IE22
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NRC USE ONLY



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/942-2920

June 3, 1983

DJS Ltr #83-541

James G. Keppler, Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Updated Reportable Occurrence Report #83-03/03X-1, Docket #050-249 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(d), abnormal degradation of systems other than those specified in item B.1.e. above designed to contain radioactive material resulting from the fission process. This revised report provides additional information as to the probable cause of the event and the corrective action taken.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC, Document Management Branch
File/NRC

JUN 20 1983

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1E92

SUPPLEMENT TO DVR

DVR NO.	STA	UNIT.	YEAR	NO.
D - 12	- 3	- 83	- 3	

PART 1	TITLE OF EVENT	OCCURRED	
	Reactor Water Cleanup Pipe Leak	1/5/83	1735
	REASON FOR SUPPLEMENTAL REPORT	DATE	TIME
	To submit additional information.		
PART 2			
	ACCEPTANCE BY STATION REVIEW	<i>f. K. K. for TBS 6/8/83 J.C. Costa</i>	
	DATE	6-13-83	
	SUPPLEMENTAL REPORT APPROVED AND AUTHORIZED FOR DISTRIBUTION	<i>Douglas Kent</i>	<i>6/14/83</i>
		STATION SUPERINTENDENT	DATE

JUN 20 1983