

## LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

/0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)  
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/6/0/3/8/3/ (8) /0/6/1/5/8/3/ (9)  
 SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On June 3, 1983, with Unit 1 at 100% power, Fire Door S71-19, between the 1H /  
 /0/3/ / Emergency Diesel Room and the Turbine Building would not close without assis- /  
 /0/4/ / tance. Since a fire watch was posted in accordance with T.S. 3.7.15 the public /  
 /0/5/ / health and safety were not affected. The degradation of fire doors between the /  
 /0/6/ / Emergency Diesel Rooms and the Turbine Building due to differential pressure /  
 /0/7/ / between the areas during diesel operation is a recurring event. This event is /  
 /0/8/ / reportable pursuant to T.S. 6.9.1.9.b. /

SYSTEM CAUSE CAUSE COMP. VALVE  
 CODE CODE SUBCODE COMPONENT CODE SUBCODE SUBCODE

/0/9/ /A/B/ (11) /E/ (12) /B/ (13) /X/X/X/X/X/X/ (14) /Z/ (15) /Z/ (16)  
 LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION  
 (17) REPORT NO. NO.  
 NUMBER /8/3/ /- /0/4/0/ / /0/3/ /L/ /- /0/

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT  
 TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER

/E/ (18) /X/ (19) /Z/ (20) /Z/ (21) /0/0/0/0/ (22) /N/ (23) /Y/ (24) /A/ (25) /C/1/7/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / Fire Door S71-18 would not self close because the reclosure mechanism had worked /  
 /1/1/ / its way out of adjustment. The mechanism was readjusted and door operation test- /  
 /1/2/ / ed satisfactorily. Design modifications to the fire doors between the Diesel /  
 /1/3/ / Rooms and the Turbine Building are being investigated. /  
 /1/4/ /

FACILITY STATUS %POWER OTHER STATUS (30) METHOD OF DISCOVERY DISCOVERY DESCRIPTION (32)

/1/5/ /E/ (28) /1/0/0/ (29) / NA / /A/ (31) / Operator Observation /

ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36)

/1/6/ /Z/ (33) /Z/ (34) / NA / / NA /

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)

/1/7/ /0/0/0/ (37) /Z/ (38) / NA /

PERSONNEL INJURIES NUMBER DESCRIPTION (41)

/1/8/ /0/0/0/ (40) / NA /

LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION

/1/9/ /Z/ (42) / NA /

PUBLICITY

ISSUED DESCRIPTION (45)

/2/0/ /N/ (44) / NA /

NAME OF PREPARER E. Wayne Harrell

PHONE (703) 894-5151

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# Vepco

VIRGINIA ELECTRIC AND POWER COMPANY

NORTH ANNA POWER STATION

P. O. BOX 402

MINERAL, VIRGINIA 23117

June 15, 1983

83 JUN 20 AIO: 01

USNRC REGION III  
ATLANTA, GEORGIA

Mr. James P. O'Reilly, Regional Administrator  
U. S. Nuclear Regulatory Commission  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, Georgia 30303

Serial No. N-83-083  
NO/WFS: dus  
Docket No. 50-338  
License No. NPF-4

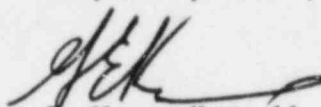
Dear Mr. O'Reilly:

Pursuant to North Anna Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No.	Applicable Technical Specifications
LER 83-040/03L-0	T.S. 6.9.1.9.b

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,

  
E. Wayne Harrell  
for Station Manager

Enclosures (3 copies)

cc: Document Control Desk (1 copy)  
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U.S. Nuclear Regulatory Commission  
Washington, D. C. 20555

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