

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

SYSTEM
CODE

CAUSE

CAUSE
SUBCODE

COMPONENT CODE

COMP.
SUBCODE

VALVE
SUBCODE

17 LER/R
REPORT
NUMBER

EVENT YEAR

SEQUENTIAL
REPORT NO.

OCCURRENCE
CODE

REPORT
TYPEREVISION
NO.

**ACTION
TAKEN**

FUTURE

EFFECT

SHUTDO

28

ATTACH

NPRO-4

31
PRIME COMP32
COMPONENT (20)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

FACILITY

% POWER

OTHER STATUS

METHOD OF DISCOVERY

DISCOVERY DESCRIPTION

ACTIVITY CONTENT
RELEASED OF RELEASE

AMOUNT OF ACTIVITY

LOCATION OF RELEASE (36)

PERSONNEL EXPOSURES	
NUMBER	TYPE
1	1
2	2
3	3
4	4
5	5
6	6
7	7
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92	92
93	93
94	94
95	95
96	96
97	97
98	98
99	99
100	100

DESCRIPTION (39)

PERSONNEL INJURIES	
NUMBER	DESCRIPTION
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OPTION (41)

LOSS OF OR DAMAGE TO FACILITY	
TYPE	DESCRIPTION
1	1.1
2	2.1
3	3.1
4	4.1
5	5.1
6	6.1
7	7.1
8	8.1
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10	10.1
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100	100.1

43

PUBLICITY DESCRIPTION (45)

NRC USE ONLY

BUONE (205) 729-0845

LER SUPPLEMENTAL INFORMATION

BFRO-50- 260/ 83024 Technical Specification Involved 3.6.F.1 & 2

Reported Under Technical Specification 6.7.2.b.(2) Date Due NRC 6/15/83

Event Narrative:

With unit 1 in a refueling outage, unit 2 at 85-percent power and unit 3 at 99-percent power, recirculation pump 2A tripped at 0150 May 15, 1983. Investigation showed that isolation valve 2-24-744 had failed due to sheared coupling pins and went closed, preventing adequate raw cooling water supply to the fluid drive 2A oil coolers. Isolation valve 2-24-744 is a Crane Model XP-3600 gate valve. The loss of cooling resulted in MG set 2A tripping on high temperature, which then tripped recirculation pump 2A. An adequate cooling water flow path was reestablished and recirculation pump 2A placed back in service at 0220 May 15, 1983, after performing SI 4.6.A.6, SI 4.6.A.7, and 4.6.E-1. Technical Specification 3.6.F.1 permits reactor operation for up to 24 hours with one recirculation pump out of service, thus making this event reportable under TS 6.7.2.b.(2). This event is considered to be a random occurrence and no further recurrence control is required.

* Previous Similar Events:

None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor

*Revision: JRP

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401

1750 Chestnut Street Tower II

June 14, 1983

83 JUN 21 9:42
USNRC REGION 1
ATLANTA, GEORGIA

Mr. James P. O'Reilly, Director
U.S. Nuclear Regulatory Commission
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 2 - DOCKET
NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE
REPORT BFRO-50-260-83024

The enclosed report provides details concerning a recirculation pump
that tripped because of inadequate cooling flow to the fluid drive oil
coolers. This report is submitted in accordance with Browns Ferry
unit 2 Technical Specification 6.7.2.b(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

H. J. Green

H. J. Green
Director of Nuclear Power

Enclosure

cc (Enclosure):

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Records Center
Institute of Nuclear Power Operations
Suite 1500
1100 Circle 75 Parkway
Atlanta, Georgia 30339

NRC Inspector, Browns Ferry

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