

LICENSEE EVENT REPORT

CONTROL BLOCK: 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 I L D R S 3 2 0 0 - 0 0 0 0 0 0 0 0 3 4 1 1 1 1 4 5
7 8 9 14 15 25 26 30 37 CAT 38
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE JO CAT 38

CON'T
0 1 REPORT SOURCE L 6 0 5 0 0 0 2 4 9 7 0 5 2 1 8 3 3 0 6 1 3 8 3 9
7 8 9 60 61 68 69 74 75 80
 DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

0 2 During normal operations, CRD scram testing was in progress. When CRD G-6 (26-23)
0 3 was pulled from the core after being scrambled, operator applied additional with-
0 4 draw signal as required. The overtravel alarm came up and position indication was
0 5 lost, indicating uncoupling of the control blade from the drive. Minimal safety
0 6 significance since all fuel limits and shutdown margin were maintained. No effect
0 7 on public health or safety. Previous occurrences on Unit 3 this cycle were
0 8 reported by RO 82-35 and 82-21 (Docket 50-249).
7 8 9 80

0 9 SYSTEM CODE R B 11 CAUSE CODE E 12 CAUSE SUBCODE B 13 COMPONENT CODE C R D R V E 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16
7 8 9 10 11 12 13 18 19 20
17 LER RO REPORT NUMBER 8 3 EVENT YEAR 8 3 SEQUENTIAL REPORT NO. 0 2 0 OCCURRENCE CODE 0 3 REPORT TYPE L REVISION NO. 0
21 22 23 24 26 27 28 29 30 31 32
 ACTION TAKEN X 18 FUTURE ACTION C 19 EFFECT ON PLANT Z 20 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 ATTACHMENT SUBMITTED N 23 NPRO-4 FORM SUB. Y 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER G 0 8 0 26
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

1 0 Probable cause is the CRD inner filter becoming detached. Drive was immediately
1 1 recoupled. Caution tag was placed to check for nuclear response whenever drive
1 2 is moved. Drive will be replaced during next refueling outage. Weekly CRD
1 3 exercising, scram testing each 16 weeks, and CRD inner filter pull tests during
1 4 rebuilding will continue.
7 8 9 80

1 5 FACILITY STATUS E 28 % POWER 0 5 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Routine Surveillance 32
7 8 9 10 12 13 44 45 46 80
1 6 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36
7 8 9 10 11 44 45 80

1 7 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39
7 8 9 11 12 13 80

1 8 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41
7 8 9 11 12 80

1 9 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43
7 8 9 10 80

2 0 PUBLICITY ISSUED N 44 DESCRIPTION N/A 45
7 8 9 10 80

8306270083 830613
 PDR ADOCK 05000249
 S PDR

NAME OF PREPARER Robert J. Whalen

PHONE (81 5) 942-2920 x523

NRC USE ONLY



Commonwealth Edison

Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

June 13, 1983

DJS Ltr #83-596

James G. Keppler, Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #83-20/03L-0, Docket #050-249 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(b), conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC, Document Management Branch
File/NRC

JUN 20 1983

1E22

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Commonwealth Edison

DEVIATION REPORT

DVR NO. 12 - 3 - 83 - 36
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION

Control Rod Drive G-6 (26-23) Uncoupled

OCCURRED

5/21/83

0833

DATE

TIME

SYSTEM AFFECTED 0300

PLANT STATUS AT TIME OF EVENT

Control Rod Drives

MODE Run

PWR(MWT) 1250

LOAD(MWE) 380

TESTING

☒ YES☐ NO

DESCRIPTION OF EVENT

When control rod drove G-6 was being withdrawn after scram

testing per DTS 300-2, overtravel was observed. Drive was immediately recoupled

per procedure DOA 300-5.

10 CFR50.72 NRC RED PHONE

☐☒

NOTIFICATION MADE

YES

NO

EQUIPMENT FAILURE

To be written

at refueling

☒ YES☐ NO

WORK REQUEST NO.

RESPONSIBLE SUPERVISOR R. Whalen

DATE 5/21/83

PART 2 OPERATING ENGINEER'S COMMENTS

The CRD was successfully recoupled per procedure DOA 300-5. The CRD

was identified on the full core display by a tag which requires verification that

the control rod is following the CRD by observing a response in the nuclear instrumenta-
tion each time the control rod is moved. The CRD will be scheduled for replacement

during the next refueling outage.

☐ EVENT OF PUBLIC INTEREST☐ TECH. SPEC. VIOLATION☐ NON REPORTABLE OCCURRENCE☐ 14 DAY REPORTABLE/T.S.☒ 30 DAY REPORTABLE/T.S. 6.6.B.2.b☐ ANNUAL/SPECL REPORT REQ'D☐ 24-HOUR NRC NOTIFICATION REQ'D

TELEPH

N/A

REGION III

DATE

TIME

TELEGM/TELECOPY

N/A

REGION III

DATE

TIME

☐ CECO CORPORATE NOTIFICATION MADE

IF ABOVE NOTIFICATION IS PER 10CFR21

☐ 5-DAY WRITTEN REPORT REQ'D PER 10CFR21

Resent

5/23/83

1626

Teletype

Dennis P. Galle

5/23/83

1530

TELETYPE

CECO CORPORATE OFFICER

DATE

TIME

PRELIMINARY REPORT
COMPLETED AND REVIEWED

T.A. Ciesla

5/23/83

OPERATING ENGINEER

DATE

INVESTIGATED REPORT & RESOLUTION
ACCEPTED BY STATION REVIEWJ. Brunner
6/14/83T.A. Ciesla
6/16/83RESOLUTION APPROVED AND
AUTHORIZED FOR DISTRIBUTION

STATION SUPERINTENDENT

6/16/83
DATE