NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1	LICENSEE CODE: V A S P S 1 (2) LICENSE NUMBER: 0 0 - 0 0 0 0 0 - 0 0 (3) LICENSE TYPE: 4 1 1 1 1 (4) CAT (5)
CON'T 0 1	REPORT SOURCE: L (6) DOCKET NUMBER: 0 5 0 0 0 2 8 0 (7) EVENT DATE: 0 5 1 7 8 3 (8) REPORT DATE: 0 6 1 6 8 3 (9)

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 On May 17, 1983, with unit at CSD, 'B' RHR pump was removed from service, on two
0 3 occassions, due to cavitation. This resulted in less than two operable RHR loops
0 4 and no loops in operation. This is contrary to T. S. 3.1.A.1.d and reportable
0 5 per T. S. 6.6.2.b(2). The cold water used to increase RCS level maintained
0 6 RCS temperature, therefore the health and safety of the public were not affected.
0 7
0 8

0 9	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
	C F (11)	E (12)	X (13)	I N S T R U (14)	I (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 3	—	0 2 4	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
E (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	L (25)	Z Z Z Z (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 An inaccurate standpipe level indication resulted in the cavitation of 'B' RHR pump.
1 1 RCS level was increased, RHR pump vented and returned to service. Adjustments were
1 2 made to the standpipe level indicator.
1 3
1 4

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	G (28)	0 0 0 (29)	CSD	A (31)	Operator Observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	N/A	N/A

	PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	N/A

	PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	N/A

	LOSS OF OR DAMAGE TO FACILITY (43) TYPE	DESCRIPTION
1 9	Z (42)	N/A

	PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
2 0	N (44)	N/A	

IE22

NAME OF PREPARER J. L. Wilson PHONE (804) 357-3184

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

JUN 16 1983

Serial No: 83-044
Docket No: 50-280
License No: DPR-32

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

Report Number	Applicable Technical Specification
83-024/03L-0	6.6.2.b (2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

DESIGNATED ORIGINAL

Certified By CHB

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ATTACHMENT 1
SURRY POWER STATION, UNIT NO. 1
DOCKET NO: 50-280
REPORT NO: 83-024/03L-0
EVENT DATE: 05-17-83

TITLE OF THE EVENT: RHR INOPERABLE

1. Description of the Event

On 05-17-83, at 0948, with the unit at CSD and drained to mid-nozzle, residual heat removal pump 1-RH-P-1B motor amperes were found to be oscillating. The pump was temporarily removed from service, vented, and returned to service. The total time that RHR was out of service was 1 hour and 21 minutes. Later, at 1328, 1-RH-P-1B was found again to be cavitating. The pump was temporarily removed from service, vented, and returned to service. The total time that RHR was out of service was 35 minutes. Both events are contrary to Technical Specification 3.1.A.1.d and reportable in accordance with T.S. 6.6.2.b(2).

2. Probable Consequences and Status of Redundant Equipment

A single reactor coolant loop or residual heat removal loop provides sufficient capacity for removal of decay heat.

With the loops drained to mid-nozzle, and thereby unavailable for heat removal, and the 'B' RH pump cavitating, only one RHR loop was possibly operable. RHR temperature was 86°F prior to the event and was still at 86° when RHR pump was returned to service.

Since a charging pump was operating and filling the reactor coolant system from the RWST, heat was being removed from the core. The health and safety of the public were not affected.

3. Cause

Cavitation of 'B' RHR pump was caused by the reactor coolant level dropping below the required NPSH. An inaccurate standpipe level caused the system to be operated in this condition.

4. Immediate Corrective Action

The B pump was vented and returned to service.

5. Subsequent Corrective Action

The erroneous standpipe level was temporarily corrected.

6. Action Taken to Prevent Recurrence

The standpipe level indication will be permanently repaired. Consideration will be given to an improved level indication system.

7. Generic Implications

None.