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August 12, 1994
C311-94-2110

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Three Mile Island Nuclear Station, Unit I (TMI-1)
Operating License No. DPR-50
Docket No. 50-289
Monthly Operating Report for July 1994

Enclosed are two copies of the July 1994 Monthly Operating Report for Three Mile Island Nuclear Station, Unit 1.

Sincerely,

T. G. Broughton
Vice President and Director, TMI

WGH

Attachments

cc: Administrator, Region I
TMI Senior Resident Inspector
T94001

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PDR ADOCK 05000289
R PDR

GPU Nuclear Corporation is a subsidiary of General Public Utilities Corporation

JE24.1

OPERATIONS SUMMARY
July 1994

The plant entered the month operating at 100% power and remained at that level through the entire month. Unit electrical output averaged approximately 788 MWe during July.

MAJOR SAFETY RELATED MAINTENANCE

The following is a summary of major safety related maintenance items accomplished during the month.

Boric Acid Supply To Bleed Tank Isolation Valve WDL-V-62

As a result leakage past Waste Disposal Liquid system valve WDL-V-62, the valve was removed from service for repair. Inspection of the disassembled valve components found the diaphragm finger plate and the gasket to be corroded. WDL-V-62 was rebuilt using a new diaphragm, diaphragm finger plate, gasket, and new studs and nuts. The valve was tested with no leakage evident and returned to service.

Pressurizer Heater Bank #5

The fault detection circuit for Pressurizer Heater Bank #5 was removed from service after it initiated numerous erroneous alarms. Troubleshooting of the system revealed failed capacitors which were replaced. All retests were performed satisfactorily and the detector returned to service.

Position Transmitter FW-ZT-1120

Feedwater Control Valve FW-V-17B position transmitter FW-ZT-1120 was removed from service due to intermittent failure. Troubleshooting identified the problem to be a failed rheostat. The rheostat was replaced and FW-ZT-1120 was returned to service.

NR-V-13D Piping

A pinhole leak was found in the 12" pipe upstream of Nuclear Service Closed Cooler NS-C-1D backwash valve NR-V-13D. Ultrasonic testing (UT) of the pipe revealed only localized wall thinning adjacent to the through wall leak. No further wall thinning was identified. The through wall leak was repaired by welding a half coupling over the defect area and installing a pipe plug in the coupling. A hydrostatic test of the repair will be performed in August.

OPERATING DATA REPORT

OPERATING STATUS

DOCKET NO. 50-289
 DATE _____
 COMPLETED BY W G HEYSEK
 TELEPHONE (717) 948-8191

1. UNIT NAME: THREE MILE ISLAND UNIT 1
 2. REPORTING PERIOD: JULY 1994
 3. LICENSED THERMAL POWER: 2568
 4. NAMEPLATE RATING (GROSS MWe): 871
 5. DESIGN ELECTRICAL RATING (NET MWe): 819
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 834
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 786

NOTES:

8. IF CHANGES OCCUR IN (ITEMS 3-7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

		THIS MONTH	YR-TO-DATE	CUMMULATIVE
11. HOURS IN REPORTING PERIOD	(HRS)	744.0	5087.0	174552.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	(HRS)	744.0	4689.7	97915.7
13. REACTOR RESERVE SHUTDOWN HOURS	(HRS)	0.0	-0.0	2284.0
14. HOURS GENERATOR ON-LINE	(HRS)	744.0	4679.2	96783.9
15. UNIT RESERVE SHUTDOWN HOURS	(HRS)	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED	(MWH)	1907510	11785887	236652981
17. GROSS ELECTRICAL ENERGY GENERATED	(MWH)	620897	3917199	79603982
18. NET ELECTRICAL ENERGY GENERATED	(MWH)	586117	3683393	74747912
19. UNIT SERVICE FACTOR	(%)	100.0	92.0	55.4
20. UNIT AVAILABILITY FACTOR	(%)	100.0	92.0	55.4
21. UNIT CAPACITY FACTOR (USING MDC NET)		100.2	92.1	54.5
22. UNIT CAPACITY FACTOR (USING DER NFT)		96.2	88.4	52.3
23. UNIT FORCED OUTAGE RATE	(%)	0.0	0.0	38.5
UNIT FORCED OUTAGE HOURS	(HRS)	0.0	0.0	60759.4
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE AND DURATION OF EACH):				

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
UNIT TMI-1
DATE
COMPLETED BY W G HEYSEK
TELEPHONE (717) 948-8191

MONTH: JULY

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
-----	--

1	796
2	793
3	789
4	789
5	787
6	782
7	785
8	775
9	780
10	787
11	798
12	794
13	787
14	787
15	785
16	788

DAY	AVERAGE DAILY POWER LEVEL (MWe-NET)
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17	791
18	788
19	785
20	780
21	781
22	783
23	786
24	786
25	790
26	790
27	793
28	796
29	793
30	788
31	790

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July 1994

DOCKET NO. 50-289
 UNIT NAME TMI-1
 DATE
 COMPLETED BY W. G. Heysek
 TELEPHONE (717) 948-8191

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report#	System Code " & "	Component Code " & "	Cause & Corrective Action to Prevent Recurrence
						None			

¹
 F Forced
 S Scheduled

²
 Reason
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & Licensing Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Other (Explain)

⁴
 Exhibit G - Instructions for
 preparation of Data Entry Sheets
 for Licensee Event Report (LER)
 File (NUREG-0161)

⁵ Exhibit I same source

⁶ Actually used exhibits F & H NUREG 0161

REFUELING INFORMATION REQUEST

1. Name of Facility: Three Mile Island Nuclear Station, Unit 1
2. Scheduled date for next refueling shutdown: September 8, 1995
3. Scheduled date for restart following current refueling: NA
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? NO
5. Scheduled date(s) for submitting proposed licensing action and supporting information: NA
6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
 - a) TMI will use the new Mark B10 fuel assembly in the Cycle 11 reload batch which is an upgraded design of the Mark B9 assembly used in Cycle 10. The Mark B10 provides a leaf-type cruciform assembly hold-down spring to replace the previous coil spring design which has experienced random failures during operation and requires visual inspection each outage. The Mark B10 design meets all current BWFC fuel design criteria and is in use at other B&W 177 FA plants.
7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool: (a) 177 (b) 601
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 1990. Phase I of the reracking project to increase spent fuel pool storage capacity permits storage of 1342 assemblies. Upon completion of Phase II of the reracking project, the full licensed capacity will be attained.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Completion of Phase I of the reracking project permits full core off-load (177 fuel assemblies) through the end of Cycle 14 and on completion of the rerack project full core off-load is assured through the end of the current operating license and beyond.