

LICENSEE EVENT REPORT

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 V A S P S 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5
8 9 14 15 25 26 30 37 38 39 40 41 42 43 44 45 46 47 48 49 50
LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CONT
 01 REPORT SOURCE L 6 0 5 0 0 0 2 8 0 7 0 5 1 3 8 3 8 0 6 1 0 8 3 9
60 61 68 69 74 75 80
DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On May 13, 1983 with Unit 1 at intermediate shutdown conditions, Steam Generator
 03 blowdown was being released to the discharge canal without monitoring the release.
 04 This is contrary to T.S.3.11.A.6 and is reportable per T.S.-6.6.2.b.(2). Primary
 05 to secondary leakage did not exist and circulating water tunnel radiation monitor
 06 was in service. Therefore, the health and safety of the public were not affected.
 07
 08

09 SYSTEM CODE M C 11 CAUSE CODE D 12 CAUSE SUBCODE Z 13 COMPONENT CODE Z Z Z Z Z Z 14 COMP. SUBCODE Z 15 VALVE SUBCODE Z 16
9 10 11 12 13 14 15 16 17 18 19 20
 17 LER/RO REPORT NUMBER 8 3 21 EVENT YEAR 8 3 22 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 22 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB. N 24 PRIME COMP. SUPPLIER A 25 COMPONENT MANUFACTURER Z Z Z Z 26
21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47
ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Procedures did not address the verification of flow through the steam generator
 11 radiation monitor. Blowdown release to the discharge canal terminated, thereby
 12 eliminating the need to monitor blowdown.
 13
 14

15 FACILITY STATUS C 28 % POWER 0 0 0 29 OTHER STATUS N/A 30 METHOD OF DISCOVERY A 31 DISCOVERY DESCRIPTION Operator Observation 32
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

16 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY N/A 35 LOCATION OF RELEASE N/A 36
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

17 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION N/A 39
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

18 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION N/A 41
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

19 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION N/A 43
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

20 PUBLICITY ISSUED N 44 DESCRIPTION N/A 45
8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

N/A
 8306210285 830610
 PDR ADOCK 05000280
 S PDR

NRC USE ONLY

NAME OF PREPARER J. L. Wilson

PHONE (804) 357-3184

ATTACHMENT 1

SURRY POWER STATION, UNIT NO. 1

DOCKET NO: 50-280

REPORT NO: 83-023/03L-0

EVENT DATE: 05-13-83

TITLE OF THE EVENT: UNMONITORED S/G BLOWDOWN

1. Description of the Event

During Unit 1 startup with the unit at intermediate shutdown conditions and steam generator blowdown in operation, operators noted "A" S.G. blowdown rad monitor isolated and the "B" & "C" S.G. blowdown rad monitor flow indicators indicating zero flow. At this same time, S.G. blowdown was being discharged to the "B" and "D" circulating water boxes. This event is contrary to Technical Specification 3.11.A.6 and is reportable per Technical Specification 6.6.2.b.(2).

2. Probable Consequences and Status of Redundant Equipment

The circulating water discharge tunnel radiation monitor and recorder remained in operation throughout the event and no increase in activity was noted. In addition, there was no primary to secondary leakage and S/G blowdown samples showed no activity in the S/G's. Therefore, the health and safety of the public was not affected.

3. Cause of the Event

The valve line up completed prior to start up calls for the b/d sample valves to be throttled. It is therefore believed that when the blowdown rad monitoring system was placed in service during the startup, the rad monitor flow isolation valves were not properly adjusted to allow for adequate blowdown flow through the rad monitors at intermediate shutdown conditions.

4. Immediate Corrective Action

Blowdown flow to the waterboxes was secured, S.G. blowdown flow to the rad monitors was established and Health Physics was notified and blowdown samples were taken and analyzed.

5. Subsequent Corrective Action

None.

6. Action Taken to Prevent Recurrence

The startup checklist for steam generator blowdown will be modified to insure that when checking blowdown rad monitor lineup, flow indication will also be monitored and noted. Also, a procedure will be developed for S.G. draining to insure proper valve lineup following the operations.

7. Generic Implications

None.

Vepco

USNRC REGIONAL OFFICE
ATLANTA, GEORGIA

VIRGINIA ELECTRIC AND POWER COMPANY
Surry Power Station
P. O. Box 315
Surry, Virginia 23883

83 JUN 16

P 8:50

JUN 10 1983

Serial No: 83-042

Docket No: 50-280

License No: DPR-32

Mr. James P. O'Reilly
Regional Administrator
Suite 2900
101 Marietta Street, NW
Atlanta, Georgia 30303

Dear Mr. O'Reilly

Pursuant to Surry Power Station Technical Specifications, the Virginia Electric and Power Company hereby submits the following Licensee Event Report for Surry Unit 1.

Report Number

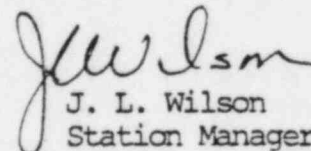
Applicable Technical Specification

83-023/03L-0

T. S. 6.6.2.b(2)

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,


J. L. Wilson
Station Manager

Enclosure

cc: Document Control Desk, USNRC
016 Phillips Bldg.
Washington, D. C. 20555

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Certified By

WB

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