

OPERATING DATA REPORT

DOCKET NO. 50 - 277

DATE JUNE 10 , 1983

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M.ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR
TELEPHONE (215) 841-5022

OPERATING STATUS

1. UNIT NAME: PEACH BOTTOM UNIT 2
2. REPORTING PERIOD: MAY, 1983
3. LICENSED THERMAL POWER (MWT): 3293
4. NAMEPLATE RATING (GROSS MWE): 1152
5. DESIGN ELECTRICAL RATING (NET MWE): 1065
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1098
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1051

NOTES: UNIT 2 EXPERIENCED THREE
FORCED SHUTDOWNS , TWO
SCHEDULED LOAD REDUCTIONS
AND TWO FORCED LOAD
REDUCTIONS.

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744	3,623	78,071
12. NUMBER OF HOURS REACTOR WAS CRITICAL	426.9	3,104.8	58,194.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	385.4	3,023.8	56,572.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1,141,555	9,585,821	166,094,793
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	365,570	3,163,190	54,683,700
18. NET ELECTRICAL ENERGY GENERATED (MWH)	350,479	3,050,497	52,435,646
19. UNIT SERVICE FACTOR	51.8	83.5	72.5
20. UNIT AVAILABILITY FACTOR	51.8	83.5	72.5
21. UNIT CAPACITY FACTOR (USING MDC NET)	44.8	80.1	63.9
22. UNIT CAPACITY FACTOR (USING DER NET)	44.2	79.1	63.1
23. UNIT FORCED OUTAGE RATE	48.2	11.2	7.9

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
REFUELING-MAINTENANCE OUTAGE BEGINNING 10/15/83 FOR 13 WEEKS.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	ACHIEVED
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

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TELEPHONE (215) 841-5022

OPERATING STATUS

1. UNIT NAME: PEACH BOTTOM UNIT 3
2. REPORTING PERIOD: MAY, 1983
3. LICENSED THERMAL POWER (MWT): 3293
4. NAMEPLATE RATING (GROSS MWE): 1152
5. DESIGN ELECTRICAL RATING (NET MWE): 1065
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1098
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1035

NOTES: UNIT 3 CONTINUED SCHEDULED
SHUTDOWN FOR REFUELING.

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):

10. REASONS FOR RESTRICTIONS, IF ANY:

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744	3,623	73,967
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0	1,039.3	54,969.0
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	0.0	1,026.3	53,627.1
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	0	2,742,355	155,957,983
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0	894,560	51,136,090
18. NET ELECTRICAL ENERGY GENERATED (MWH)	* -5,418	832,372	49,075,164
19. UNIT SERVICE FACTOR	0.0	28.3	72.5
20. UNIT AVAILABILITY FACTOR	0.0	28.3	72.5
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0	22.2	64.1
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0	21.6	62.3
23. UNIT FORCED OUTAGE RATE	0.0	2.6	7.3
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): SCHEDULED SHUTDOWN FOR REFUELING AND MAINTENANCE, STARTED 2/13/83.			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 6/30/83

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):	FORECAST	Achieved
INITIAL CRITICALITY	-----	-----
INITIAL ELECTRICITY	-----	-----
COMMERCIAL OPERATION	-----	-----

* - NEGATIVE VALUE REPORTED FOR CONSISTENCY WITH FEDERAL ENERGY REGULATORY COMMISSION REPORTS.

UNIT NAME PEACH BOTTOM UNIT 2

DATE JUNE 10 , 1983

REPORT MONTH MAY, 1983

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR
TELEPHONE (215) 841-5022

NO.	DATE	TYPE (1)	DURATION (HOURS)	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
9	830503	P	138.0	A	1	NA	CB	VALVEX	SHUTDOWN DUE TO HIGH DRYWELL PUMP-OUT RATE AND REPAIR PACKING LEAK ON RECIRC. LINE VALVE.
10	830514	S	00.0	H	4	NA	RC	ZZZZZZ	LOAD REDUCTION FOR CONTROL ROD PATTERN ADJUSTMENT.
11	830517	P	00.0	A	4	NA	HA	ELECON	LOAD REDUCTION DUE TO LOSS OF COOLING ISOPHASE BUS.
12	830519	P	00.0	H	4	NA	HH	PUMPXI	LOAD REDUCTION DUE TO "C" CONDENSATE PUMP BEING TAKEN OFF.
13	830520	P	23.6	A	3	NA	EB	TRANSF	SHUTDOWN DUE TO "B" PHASE TRANSFORMER LOCKOUT.
14	830521	P	197.0	A	1	NA	EB	TRANSF	SHUTDOWN DUE TO "B" PHASE TRANSFORMER LOCKOUT.
15	830530	S	00.0	H	4	NA	RC	ZZZZZZ	LOAD REDUCTION FOR CONTROL ROD PATTERN ADJUSTMENT.
			358.6						

(1)

(2)

(3)

(4)

P - FORCED
S - SCHEDULED

REASON

A - EQUIPMENT FAILURE (EXPLAIN)
B - MAINTENANCE OR TEST
C - REFUELING
D - REGULATORY RESTRICTION
E - OPERATOR TRAINING + LICENSE EXAMINATION
F - ADMINISTRATIVE
G - OPERATIONAL ERROR (EXPLAIN)
H - OTHER (EXPLAIN)

METHOD

1 - MANUAL
2 - MANUAL SCRAM.
3 - AUTOMATIC SCRAM.
4 - OTHER (EXPLAIN)

EXHIBIT G - INSTRUCTIONS
FOR PREPARATION OF DATA
ENTRY SHEETS FOR LICENSEE
EVENT REPORT (LER)
FILE (NUREG-0161)

(5)

EXHIBIT I - SAME SOURCE

UNIT SHUTDOWNS AND POWER REDUCTIONS

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UNIT NAME PEACH BOTTOM UNIT 3

DATE JUNE 10 , 1983

REPORT MONTH MAY, 1983

COMPLETED BY PHILADELPHIA ELECTRIC COMPANY

W.M. ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR
TELEPHONE (215) 841-5022

NO.	DATE	TYPE (1)	DURATION (HOURS)	REASON (2)	METHOD OF SHUTTING DOWN REACTOR (3)	LICENSEE EVENT REPORT #	SYSTEM CODE (4)	COMPONENT CODE (5)	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
3	830501	S	744.0	C	1	NA	RC	FUELXX	CONTINUING REFUELING OUTAGE.
			744.0						

(1)

P - FORCED
S - SCHEDULED

(2)

REASON
A - EQUIPMENT FAILURE (EXPLAIN)
B - MAINTENANCE OR TEST
C - REFUELING
D - REGULATORY RESTRICTION
E - OPERATOR TRAINING + LICENSE EXAMINATION
F - ADMINISTRATIVE
G - OPERATIONAL ERROR (EXPLAIN)
H - OTHER (EXPLAIN)

(3)

METHOD
1 - MANUAL
2 - MANUAL SCRAM.
3 - AUTOMATIC SCRAM.
4 - OTHER (EXPLAIN)

(4)

EXHIBIT G - INSTRUCTIONS
FOR PREPARATION OF DATA
ENTRY SHEETS FOR LICENSEE
EVENT REPORT (LER)
FILE (NUREG-0761)

(5)

EXHIBIT I - SAME SOURCE

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50 - 277

UNIT PEACH BOTTOM UNIT 2

DATE JUNE 10 , 1983

COMPANY PHILADELPHIA ELECTRIC COMPANY

W.M.ALDEN
ENGINEER-IN-CHARGE
LICENSING SECTION
GENERATION DIVISION-NUCLEAR

TELEPHONE (215) 841-5022

MONTH MAY 1983

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	1065	17	902
2	1063	18	1065
3	528	19	1055
4	0	20	349
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	268	25	0
10	827	26	0
11	961	27	0
12	975	28	0
13	969	29	218
14	696	30	820
15	999	31	888
16	1061		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50 - 278

UNIT PEACH BOTTOM UNIT 3

DATE JUNE 10 , 1983

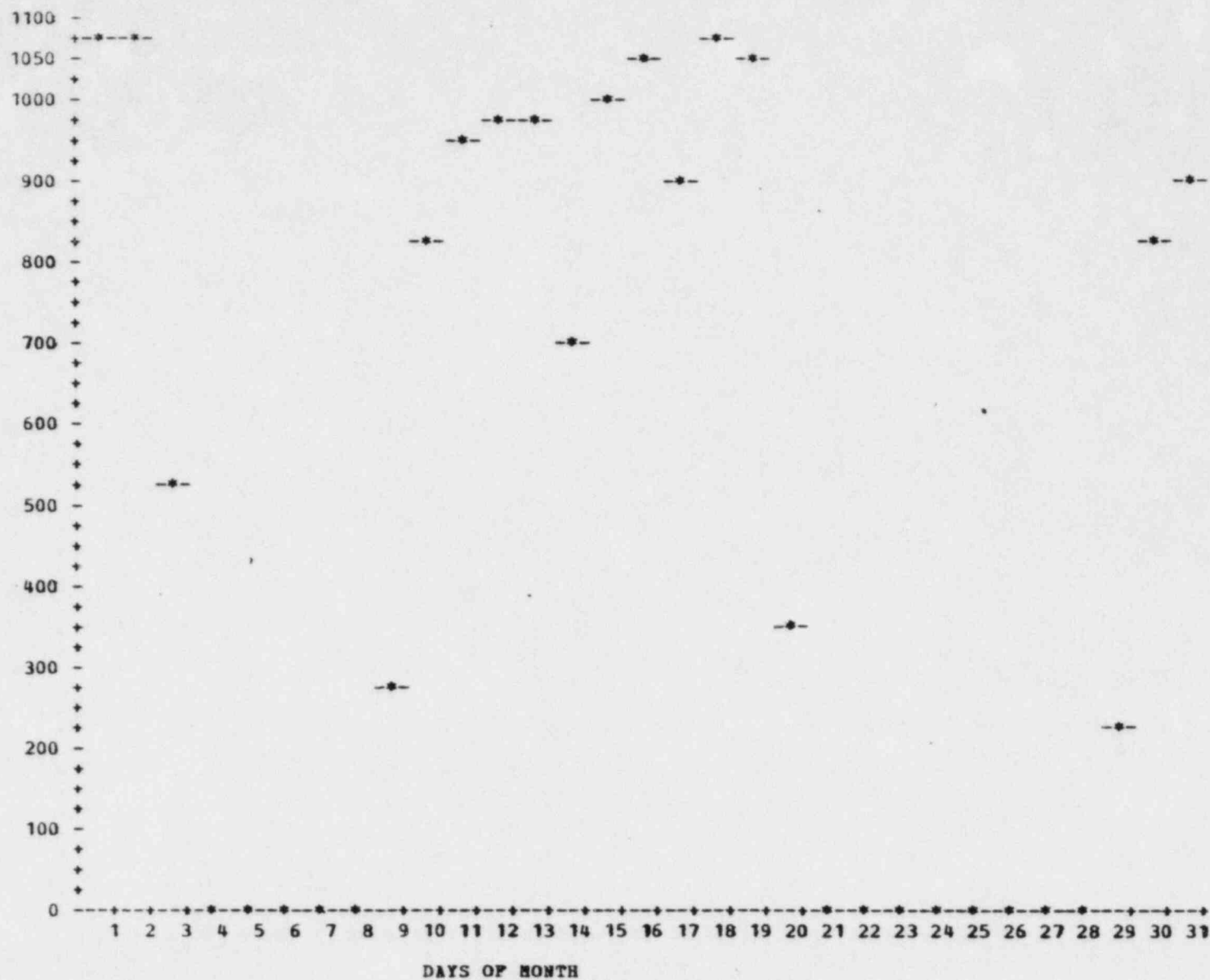
COMPANY PHILADELPHIA ELECTRIC COMPANY

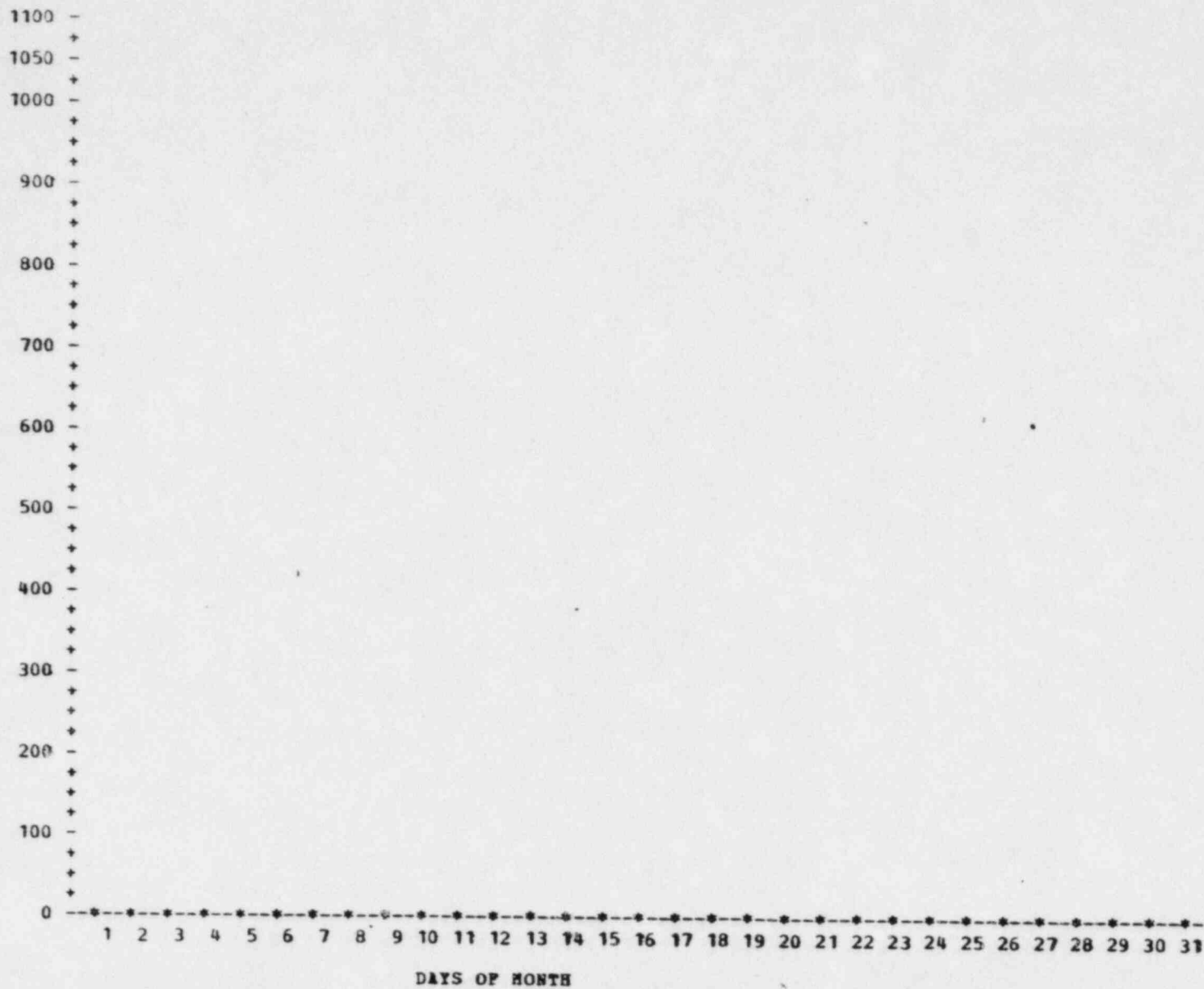
W.M.ALDEN
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LICENSING SECTION
GENERATION DIVISION-NUCLEAR

TELEPHONE (215) 841-5022

MONTH MAY 1983

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

AVERAGE DAILY POWER LEVEL
(MWE-NET)

AVERAGE DAILY POWER LEVEL
(MWE-NET)

REFUELING INFORMATION

1. Name of facility:

Peach Bottom Unit 2

2. Scheduled date for next refueling shutdown:

October 15, 1983

3. Scheduled date for restart following refueling:

January 14, 1984

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

If answer is yes, what, in general, will these be?

Technical Specifications to accommodate reload fuel.
Modifications to reactor core operating limits are expected.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

September 30, 1983

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

None expected

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

(a) Core - 764 Fuel Assemblies

(b) Fuel Pool - 1170 Fuel Assemblies, 58 Fuel Rods

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The spent fuel pool storage capacity has been relicensed for 2316 fuel assemblies.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

March, 1990 (September, 1985 with reserve for full core discharge)

REFUELING INFORMATION

1. Name of facility:

Peach Bottom Unit 3

2. Scheduled date for next refueling shutdown:

February 13, 1983

3. Scheduled date for restart following refueling:

June 30, 1983

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Yes

If answer is yes, what, in general, will these be?

Technical Specifications to accommodate reload fuel.
Modifications to reactor core operating limits are expected.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

December 30, 1982

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

None expected

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool:

- (a) Core - 764 Fuel Assemblies
- (b) Fuel Pool - 1212 Fuel Assemblies, 6 Fuel Rods

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The spent fuel pool storage capacity has been relicensed for 2816 fuel assemblies.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

September, 1990 (March, 1986 with reserve for fuel core discharge)

Peach Bottom Atomic Power Station
Narrative Summary of Operating Experiences
May, 1983

Unit 2:

The unit began the month at full power and was shutdown on May 3 to repair a packing leak on the 'A' recirculation loop isolation valve. During this outage, in light of the recirc crack indication findings in Unit 3, the decision was made to examine certain welds in Unit 2. The selection of the welds to be inspected was made by choosing a representative sample of welds in Unit 2 that were typical of the welds found with indications in Unit 3. Three welds were chosen to be inspected. The intermittent circumferential crack was formed in the RHR shutdown cooling suction line. "Water Sensitive Tape" was installed on 10 selected welds in Unit 2 to improve early warning detection. Following consultation with the NRC staff the unit returned to service on May 9 on a ramp to full power, which was achieved on May 16. On May 17 load was reduced to 600 MWe due to an after-cooler leak on the Turbine Building Cooling Water System which caused a loss of cooling to the isophase bus. On May 19, load was reduced to 890 MWe to remove the 'C' condensate pump from service to repair a cooling water line.

The unit scrambled on generator lockout on May 20 due to a sudden pressure relay operation on the B phase transformer. The relay operation was caused by an arc due to a faulty electrical connection on the primary side of the transformer. The unit returned to service on May 29 ending the month on a ramp to full power.

Unit 3:

The unit continued through the month in its fifth refueling outage. The recirc M-G sets have been reassembled following inspection of bearings and fluid drives. Modifications to install diverse level instrumentation on the Scram Discharge Volume were completed during the month.

While performing non-destructive examinations following Induction Heating Stress Improvement Treatment (IHST) of Reactor Recirculation and RHR suction lines, cracks were found in the heat affected zone of 15 welds. IHST was performed on 91 welds in the Recirc & RHR piping. Five of the welds are in the 20" non-isolatable section of the RHR suction line. The remaining ten welds with cracks are the vertical riser to elbow welds in all of the ten 12" recirculation risers. Weld overlay repair is being performed on the affected welds.

PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

June 15, 1983

Docket Nos. 50-277
50-278

Director
Office of Inspection & Enforcement
US Nuclear Regulatory Commission
Washington, DC 20555

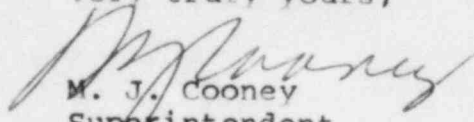
Attention: Document Control Desk

SUBJECT: Peach Bottom Atomic Power Station
Monthly Operating Report

Gentlemen:

Attached are twelve copies of the monthly operating report for Peach Bottom Units 2 and 3 for the month of May, 1983 forwarded pursuant to Technical Specification 6.9.1.C under the guidance of Regulatory Guide 10.1, Revision 4.

Very truly yours,



M. J. Cooney
Superintendent
Nuclear Generation Division

Attachment

cc: Mr. J. M. Allen, NRC
Mr. A. R. Blough, NRC Site Inspector
Mr. Stan P. Mangi, Dept. of Envir. Resources
Mr. P. A. Ross, NRC - INPO Records Center

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