

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

O. W. DIXON, JR.  
VICE PRESIDENT  
NUCLEAR OPERATIONS

June 8, 1983

Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Virgil C. Summer Nuclear Station  
Docket No. 50/395  
Operating License No. NPF-12  
Reactor Trip Breakers

Dear Mr. Denton:

On May 26, 1983 representatives from the South Carolina Electric and Gas Company (SCE&G) met with the NRC Staff to discuss issuance of the NRC's Safety Evaluation Report (SER) for the Reactor Trip Breaker issue for the Virgil C. Summer Nuclear Station. As a result of these discussions, agreement was reached on the following items.

1. SCE&G will revise the current maintenance procedures to ensure that when performing maintenance on the reactor trip and bypass breakers that the lubricant and the cleanliness of the roller bearing in the undervoltage (UV) device is inspected. The procedure will also include a check of the dimensional tolerance of the pre- and post-travel of the trip tabs. The procedure will ensure performance before and after preventive maintenance of the following six (6) months Surveillance/Maintenance items: (a) response time testing of UV/breakers on UV simulated signal from RPS, (b) trip shaft force measurements, (c) UV output force measurements, (d) check of dimensional tolerance of pre- and post-travel of the trip tab and (e) inspection of the lubricant and cleanliness of the roller bearing on the UV device. Also, the six (6) months surveillance/maintenance items will be performed within the normal surveillance time interval specified in the Virgil C. Summer Nuclear Station Technical Specification Section 4.0.2.
2. The design changes discussed in our May 12, 1983 letter will be provided to the NRC Staff for approval prior to implementing the plant modification.

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Mr. Harold R. Denton  
Reactor Trip Breakers  
June 8, 1983  
Page #2

3. In addition to the normal reporting procedures defined in Technical Specifications, SCE&G will notify the NRC Licensing Project Manager, by phone, of any failure of the reactor trip or bypass breakers, either in service or during testing, within the first normal working day after the failure. Written documentation will be through the normal Technical Specification requirements since the Licensing Project Manager receives a copy of reports made to the Region through the normal NRC correspondence distribution procedures. Efforts will be made to preserve the evidence of the failure unless it is determined that safe operation or safe shutdown of the plant will be compromised.
4. SCE&G will submit the Life Cycle Program scope and schedule discussed in our May 12, 1983 letter to the NRC for approval prior to commencing this program.
5. SCE&G will revise plant procedure EOP-5 as follows:
  - (a) Require the operator to check decreasing reactor power as an indication of an actual reactor trip.
  - (b) Include the statement "or attempt to insert the control rods manually."
6. SCE&G will incorporate into the post trip review program the following aspects:
  - a) Definition of who is authorized to permit restart of the plant following a trip.
  - b) Guidance on what to review during the post trip review and who is responsible for performing this review.
  - c) Guidance on a default clause when the actual cause of the trip is indeterminant.

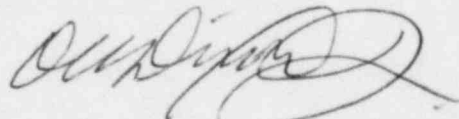
The revised procedure/program discussed in Items 5 and 6 above will be provided to the Staff by July 15, 1983. The design change package (item 2) and the Life Cycle Program information (item 4) will be submitted by August 1, 1983.

Mr. Harold R. Denton  
Reactor Trip Breakers  
June 8, 1983  
Page #2

This letter also provides confirmation that the new UV devices installed in the reactor trip and bypass breakers were operated twenty-five (25) times by Westinghouse prior to shipment to the Virgil C. Summer Nuclear Station. Documentation of the total number of times the devices have been operated will be maintained. In any future replacement of the breakers, SCE&G shall obtain from the vendor, information concerning the number of times the devices were operated for testing purposes prior to shipment.

If you have any questions, please let us know.

Very truly yours,



O. W. Dixon, Jr.

RBC:OWD/fjc

cc: V. C. Summer  
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File (Lic./Eng.)

# SOUTH CAROLINA ELECTRIC & GAS COMPANY

Inter-Office Correspondence

## NUCLEAR ENGINEERING

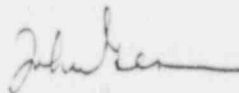
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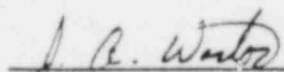
Subject V. C. Summer Nuclear Station May 27, 1983  
MRF 10516 Evacuation Alarms Inaudible CGSS-10448  
IE 000040  
NE File: 20.6600  
3,1020  
2.8925 IEB 79-16  
To M. N. Browne Attention of B. Smith

Nuclear Engineering (NE) has investigated MRF-10516 and has determined that the installation of the additional plant alarms does not have to be completed prior to reaching 100% power. NE has determined, however, that a schedule for implementation should be provided to the Nuclear Regulatory Commission on implementation of this change. To insure adequate time for procurement of hardware and to complete the necessary design implementation, completion should be scheduled by the end of first refueling.

If there are any questions, please feel free to call.

  
John Gesn

Approved By:

  
J. A. Wactor, Director  
Electrical Engineering

lkh

cc: O. W. Dixon, Jr.  
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