



**CENTERIOR
ENERGY**

PERRY NUCLEAR POWER PLANT

10 CENTER ROAD
PERRY, OHIO 44081
(216) 259-3737

Mail Address:
PO. BOX 97
PERRY, OHIO 44081

Robert A. Stratman
VICE PRESIDENT - NUCLEAR

August 8, 1994
PY-CEI/NRR-1837L

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Perry Nuclear Power Plant
Docket No. 50-440
Special Report - ECCS Injection

Gentlemen:

Perry Nuclear Power Plant Technical Specification 3.5.1 requires that a Special Report be submitted in accordance with Technical Specification 6.9.2 for Emergency Core Cooling System (ECCS) actuations and injections into the reactor coolant system. During refueling outage four, manual ECCS actuations were performed as part of pre-planned testing to support requirements for NRC Generic Letter 89-10, "Safety Related Motor Operated Valve Testing and Surveillance." The attached Special Report provides details of these actuations and meets the reporting requirements of Technical Specification 3.5.1 Action h and 6.9.2.

If you have questions or require additional information, please contact Mr. James D. Kloosterman, Manager - Regulatory Affairs at (216) 280-5833.

Very truly yours,

RAS:DAH:ns

Attachment

cc: NRC Project Manager
NRC Resident Inspector Office
NRC Region III

120089

9408160052 940808
PDR ADOCK 05000440
S PDR

Operating Companies
Cleveland Electric Illuminating
Toledo Edison

Accol
1/1

PERRY NUCLEAR POWER PLANT
NRC Docket 50-440

Special Report
ECCS Actuation and Injection into the RCS

On May 15, 1994 and June 18, 1994, Emergency Core Cooling System (ECCS) manual actuations with injection into the Reactor Coolant System (RCS) occurred as part of pre-planned activities for testing. This report is submitted in accordance with the requirements of Technical Specification 3.5.1 Action h which requires a Special Report (Technical Specification 6.9.2) to be submitted within 90 days in the event an ECCS actuates and injects water into the RCS.

During the last refueling outage (RFO4), diagnostic testing of safety related motor operated valves (MOV) was being performed in accordance with the requirements of Generic Letter (GL) 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance." This testing included dynamic flow testing of the Low Pressure Core Injection (LPCI) system train B injection valve (1E12-F042B) and the High Pressure Core Spray (HPCS) system injection valve (1E22-F004). In order to perform dynamic flow testing of these valves, the respective systems had to be operated with flow to the reactor vessel.

During testing on May 15, 1994, the reactor was defueled with the reactor pressure vessel head removed. During testing on June 18, 1994, the plant was in Operational Condition 5 (Refueling) with fuel in the vessel. Due to the plant and system conditions during this testing, these injections do not add to the number of actuation cycles to date. There have been seven previous HPCS injections and no previous LPCI system injections. The injection nozzle usage factors are less than 0.70.

Prior to the testing, courtesy notifications to the NRC Operations Center were made utilizing the Emergency Notification System (ENS). These notifications were made on May 15, 1994 at 0049 and June 18, 1994 at 2050.