

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 except that:

- a. The Radiation Protection Superintendent shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.
- b. Personnel in the Health Physics, Chemistry and Radwaste Departments shall meet or exceed the minimum qualifications of ANSI N18.1-1971.
- c. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of 10 CFR Part 55.
- d. Personnel in the Nuclear Quality Assurance Department, and other staff personnel who perform inspection, examination, and testing functions, shall meet or exceed the minimum qualifications of Regulatory Guide 1.58, Rev. 1, September 1980. (Endorses ANSI N45.2.6-1978).

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager-Nuclear and shall meet or exceed the requirements and recommendations of Section 5.2 of ANSI 3.1-1978 and 10 CFR Part 55.

6.5 REVIEW AND AUDIT

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the General Manager Plant Operations on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the following members:

Manager Technical Services
Manager Operations and Maintenance
Management Knowledgeable in Engineering*
Maintenance Superintendent
Operations Superintendent
Radiation Protection Superintendent
Management Knowledgeable in Quality Assurance/Control

Either the Manager Technical Services or the Manager Operations and Maintenance will be Chairman.

*Management knowledgeable in engineering is one of the following: STA Supervisor, Systems Engineering Electric/HVAC Supervisor, Systems Engineering Mechanical Supervisor or Reactor Engineering and Performance Supervisor.

ADMINISTRATIVE CONTROLS

ALTERNATES

6.5.1.3 In the absence of the PORC Chairman, the General Manager Plant Operations will appoint a temporary Chairman. All other alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate.

QUORUM

6.5.1.5 The quorum of the PORC necessary for the performance of the PORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three members, including alternates.

RESPONSIBILITIES

6.5.1.6 The PORC shall be responsible for the below listed activities. The PORC may delegate the performance of reviews but will maintain cognizance over the responsibility for them.

- a. Review of (1) all procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, and (3) any other proposed procedures or changes thereto as determined by the General Manager Plant Operations to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.
- e. Review of investigations of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager Plant Operations and to the Safety Review Committee.
- f. Review of all REPORTABLE EVENTS.

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- g. Review of unit operations to detect potential hazards to nuclear safety.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the General Manager Plant Operations or the Safety Review Committee.
- i. Review of the Security Plan and implementing procedures and submittal of recommended changes to the Safety Review Committee.
- j. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Safety Review Committee.
- k. Review and documentation of judgement concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PORC meeting.
- l. Review of proposed modifications to the CPC addressable constants based on information obtained through the Plant Computer-CPC data link.
- m. Review of any accidental, unplanned or uncontrolled radioactive release including reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President Operations and to the Safety Review Committee.
- n. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL, and major changes to radwaste treatment systems.
- o. Review of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Safety Review Committee.

AUTHORITY

6.5.1.7 The PORC shall:

- a. Recommend in writing to the General Manager Plant Operations, prior to implementation except as provided in Specification 6.8.3, approval or disapproval of items considered under Specification 6.5.1.6a. through d. and l.
- b. Render determinations in writing, prior to implementation except as provided in Specification 6.8.3, with regard to whether or not each item considered under Specification 6.5.1.6a. through e. constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Vice President Operations and the Safety Review Committee of disagreements between the PORC and the General Manager Plant Operations; however, the General Manager Plant Operations shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.1.8 The PORC shall maintain written minutes of each PORC meeting that, at a minimum, document the results of all PORC activities performed under the responsibility and authority provisions of these technical specifications. Copies shall be provided to the Vice President Operations and the Safety Review Committee.

6.5.2 SAFETY REVIEW COMMITTEE (SRC)

FUNCTION

6.5.2.1 The SRC shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,
- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering and
- h. Quality assurance practices.

COMPOSITION

6.5.2.2 The SRC shall be composed of at least five members, including the Chairman. Members of the SRC may be from within the LP&L organization or from organizations external to LP&L.

The qualifications of members selected for the SRC shall be in accordance with Section 4.7 of ANSI/ANS 3.1-1978.

ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in SRC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SRC Chairman to provide expert advice to the SRC.

ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

QUORUM

6.5.2.6 The quorum of the SRC necessary for the performance of the review and audit function of these technical specifications shall consist of a minimum of five members or of not less than a majority of the composition of members in Specification 6.5.2.2, whichever is greater. No more than a minority of the members shall have line responsibility for operation of the plant.

REVIEW

6.5.2.7 The SRC shall be responsible for the review of:

- a. The safety evaluations for (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment, or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- d. Proposed changes to Technical Specifications or this Operating License;
- e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- g. ALL REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and
- i. Reports and meeting minutes of the PORC.

ADMINISTRATIVE CONTROLS

AUDITS

- 6.5.2.8 Audits of unit activities shall be performed under the cognizance of the SRC. These audits shall encompass:
- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.
 - b. The performance, training, and qualifications of the entire unit staff at least once per 12 months.
 - c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.
 - d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months.
 - e. Any other area of unit operation considered appropriate by the SRC or the Vice President Operations.
 - f. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel.
 - g. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year.
 - h. The Primary Coolant Sources Outside Containment Program at least once per 24 months.
 - i. The In-Plant Radiation Monitoring Program at least once per 24 months.
 - j. The Secondary Water Chemistry Program at least once per 24 months.
 - k. The Post-Accident Sampling Program at least once per 24 months.
 - l. The Basemat Monitoring Program at least once per 24 months.
 - m. The radiological environmental monitoring program and the results thereof at least once per 12 months.
 - n. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.
 - o. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- p. The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975 at least once per 12 months.

AUTHORITY

6.5.2.9 The SRC shall report to and advise the Vice President Operations on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.

RECORDS

6.5.2.10 Records of SRC activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each SRC meeting shall be prepared, approved, and forwarded to the Vice President Operations within 14 days following each meeting.
- b. Reports of reviews encompassed by Specification 6.5.2.7 shall be prepared, approved, and forwarded to the Vice President Operations within 14 days following completion of the review.
- c. Audit reports encompassed by Specification 6.5.2.3 shall be forwarded to the Vice President Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the PORC and the results of this review shall be submitted to the SRC and the Vice President Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President Operations and the SRC shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the SRC and the Vice President Operations within 14 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978 and those required for implementing the requirements of NUREG-0737.
- b. Refueling operations.
- c. Surveillance and test activities of safety-related equipment.
- d. Security Plan implementation.
- e. Emergency Plan implementation.
- f. Fire Protection Program implementation.
- g. Modification of Core Protection Calculator (CPC) Addressable Constants, including independent verification of modified constants.

NOTES:

- (1) Modification to the CPC addressable constants based on information obtained through the Plant Computer - CPC data link shall not be made without prior approval of the PORC.
- (2) Modifications to the CPC software (including algorithm changes and changes in fuel cycle specific data) shall be performed in accordance with the most recent version of CEN-39(A)-P, "CPC Protection Algorithm Software Change Procedure," that has been determined to be applicable to the facility. Additions or deletions to CPC Addressable Constants or changes to Addressable Constant software limits values shall not be implemented without prior NRC approval.
- h. Administrative procedures implementing the overtime guidelines of Specification 6.2.2e., including provisions for documentation of deviations.
- i. PROCESS CONTROL PROGRAM implementation.

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- j. OFFSITE DOSE CALCULATION MANUAL implementation.
- k. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed as specified in Specification 6.5.1.6 and shall be approved by the General Manager Plant Operations prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected;
- c. The change is documented, reviewed by the PORC and approved by the General Manager Plant Operations within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the containment spray, safety injection, hydrogen analyzer, and the post-accident sampling system. The program shall include the following:

- 1. Preventive maintenance and periodic visual inspection requirements, and
- 2. Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- i. Records of quality assurance activities required by the QA Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PORC and the SRC.
- l. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.8 including the date at which the service life commences and associated installation and maintenance records.
- m. Records of secondary water sampling and water quality.
- n. Records of audits performed under the requirements of Specification 6.5.2.8.
- o. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- p. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/h but less than 1000 mrem/h shall be barricaded and conspicuously posted as a high radiation area and entrance hereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

*Health physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

TABLE 3.3-1 (Continued)

TABLE NOTATION

* With the protective system trip breakers in the closed position, the CEA drive system capable of CEA withdrawal, and fuel in the reactor vessel.

The provisions of Specification 3.0.4 are not applicable.

**Not applicable above $10^{-4}\%$ RATED THERMAL POWER.

- (a) Trip may be manually bypassed above $10^{-4}\%$ of RATED THERMAL POWER; bypass shall be automatically removed when THERMAL POWER is less than or equal to $10^{-4}\%$ of RATED THERMAL POWER.
- (b) Trip may be manually bypassed below 400 psia; bypass shall be automatically removed whenever pressurizer pressure is greater than or equal to 500 psia.
- (c) Trip may be manually bypassed below $10^{-4}\%$ of RATED THERMAL POWER; bypass shall be automatically removed when THERMAL POWER is greater than or equal to $10^{-4}\%$ of RATED THERMAL POWER. During testing pursuant to Special Test Exception 3.10.3, trip may be manually bypassed below 5% of RATED THERMAL POWER; bypass shall be automatically removed when THERMAL POWER is greater than or equal to 5% of RATED THERMAL POWER.
- (d) Trip may be bypassed during testing pursuant to Special Test Exception 3.10.3.
- (e) See Special Test Exception 3.10.2.
- (f) Each channel shall be comprised of two trip breakers; actual trip logic shall be one-out-of-two taken twice.
- (g) High steam generator level trip may be manually bypassed in Modes 1 and 2, at 20% power and below.

ACTION STATEMENTS

- ACTION 1 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and/or open the protective system trip breakers.
- ACTION 2 - With the number of channels OPERABLE one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may continue provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. If the inoperable channel is bypassed, the desirability of maintaining this channel in the bypassed condition shall be reviewed in accordance with Specification 6.5.1.6k. The channel shall be returned to OPERABLE status prior to STARTUP following the next COLD SHUTDOWN.

TABLE 3.3-3 (Continued)

TABLE NOTATION

- (a) Trip function may be bypassed in this MODE when pressurizer pressure is less than 400 psia; bypass shall be automatically removed when pressurizer pressure is greater than or equal to 500 psia.
- (b) An SIAS signal is first necessary to enable CSAS logic.
- * The provisions of Specification 3.0.4 are not applicable.

ACTION STATEMENTS

- ACTION 12 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- ACTION 13 - With the number of channels OPERABLE one less than the Total Number of Channels, STARTUP and/or POWER OPERATION and/or operation in the other applicable MODE(S) may continue provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. If the inoperable channel is bypassed, the desirability of maintaining this channel in the bypassed condition shall be reviewed in accordance with Specification 6.5.1.6k. The channel shall be returned to OPERABLE status no later than prior to entry into the applicable MODE(S) following the next COLD SHUTDOWN.

With a channel process measurement circuit that affects multiple functional units inoperable or in test, bypass or trip all associated functional units as listed below:

Process Measurement Circuit	Functional Unit Bypassed/Tripped
1. Containment Pressure - High	Containment Pressure - High (ESF) Containment Pressure - High (RPS)
2. Steam Generator Pressure - Low	Steam Generator Pressure - Low Steam Generator ΔP 1 and 2 (EFAS)
3. Steam Generator Level	Steam Generator Level - Low Steam Generator Level - High Steam Generator ΔP (EFAS)

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ATTACHMENT B

ADMINISTRATIVE CONTROLS

6.3 UNIT STAFF QUALIFICATIONS

6.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of ANSI/ANS 3.1-1978 except that:

- a. The Radiation Protection Superintendent shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.
- b. Personnel in the Health Physics, Chemistry and Radwaste Departments shall meet or exceed the minimum qualifications of ANSI N18.1-1971.
- c. The licensed Operators and Senior Operators shall also meet or exceed the minimum qualifications of the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees.
- d. Personnel in the Nuclear Quality Assurance Department, and other staff personnel who perform inspection, examination, and testing functions, shall meet or exceed the minimum qualifications of Regulatory Guide 1.58, Rev. 1, September 1980. (Endorses ANSI N45.2.6-1978)

6.4 TRAINING

6.4.1 A retraining and replacement training program for the unit staff shall be maintained under the direction of the Training Manager-Nuclear and shall meet or exceed the requirements and recommendations of Section 5.2 of ANSI 3.1-1978 and Appendix A of 10 CFR Part 55 and the supplemental requirements specified in Sections A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees, and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT Not used.

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

~~6.5.1.1 The PORC shall function to advise the General Manager Plant Operations on all matters to nuclear safety.~~

COMPOSITION

~~6.5.1.2 The PORC shall be composed of the following members:~~

~~Manager Technical Services
Manager Operations and Maintenance
Management Knowledgeable in Engineering*
Maintenance Superintendent
Operations Superintendent
Radiation Protection Superintendent
Management Knowledgeable in Quality Assurance/Control~~

~~Either the Manager Technical Services or the Manager Operations and Maintenance will be Chairman.~~

WATERFORD - UNIT 3

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AMENDMENT NO. 79

~~*Management knowledgeable in engineering is one of the following: STA Supervisor, Systems Engineering Electric/HVAC Supervisor, Systems Engineering Mechanical Supervisor or Reactor Engineering and Performance Supervisor.~~

ADMINISTRATIVE CONTROLS

ALTERNATES

~~6.5.1.3 In the absence of the PORC Chairman, the General Manager Plant Operations will appoint a temporary Chairman. All other alternate members shall be appointed in writing by the PORC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in PORC activities at any one time.~~

MEETING FREQUENCY

~~6.5.1.4 The PORC shall meet at least once per calendar month and as convened by the PORC Chairman or his designated alternate.~~

QUORUM

~~6.5.1.5 The quorum of the PORC necessary for the performance of the PORC responsibility and authority provisions of these Technical Specifications shall consist of the Chairman or his designated alternate and three members, including alternates.~~

RESPONSIBILITIES

~~6.5.1.6 The PORC shall be responsible for:~~

- ~~a. Review of (1) all plant administrative procedures required by Specification 6.8 and changes thereto, (2) all programs required by Specification 6.8 and changes thereto, (3) changes to the Waterford 3 Emergency Operating Procedures required to implement the requirements of NUREG 0737 and NUREG 0737, Supplement 1, as stated in Generic Letter 82-33, and (4) any other proposed procedures or changes thereto as determined by the General Manager Plant Operations.~~
- ~~b. Review of all proposed tests and experiments that affect nuclear safety.~~
- ~~c. Review of all proposed changes to Appendix "A" Technical Specifications.~~
- ~~d. Review of all proposed changes or modifications to unit systems or equipment that affect nuclear safety.~~
- ~~e. Review of investigations of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the General Manager Plant Operations and to the Safety Review Committee.~~
- ~~f. Review of all REPORTABLE EVENTS.~~

ADMINISTRATIVE CONTROLS

RESPONSIBILITIES (Continued)

- g- ~~Review of unit operations to detect potential hazards to nuclear safety.~~
- h- ~~Performance of special reviews, investigations, or analyses and reports thereon as requested by the General Manager Plant Operations or the Safety Review Committee.~~
- i- ~~Review of the Security Plan and submittal of recommended changes to the Safety Review Committee.~~
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- k- ~~Review and documentation of judgment concerning prolonged operation in bypass, channel trip, and/or repair of defective protection channels of process variables placed in bypass since the last PORC meeting.~~
- l- ~~Review of proposed modifications to the CPC addressable constants based on information obtained through the Plant Computer CPC data link.~~
- m- ~~Review of any accidental, unplanned or uncontrolled radioactive release including reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Vice President Operations and to the Safety Review Committee.~~
- n- ~~Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL, and major changes to radwaste treatment systems.~~
- o- ~~Review of the Fire Protection Program and submittal of recommended changes to the Safety Review Committee.~~
- p- ~~Review of proposed procedures and changes to procedures which involve an unreviewed safety question as defined in 10CFR50.59.~~

AUTHORITY

6.5.1.7 The PORC shall:

- a- ~~Recommend in writing to the General Manager Plant Operations, prior to implementation except as provided in Specification 6.8.3, approval or disapproval of items considered under Specification 6.5.1.6a, through d, and 1.~~
- b- ~~Render determinations in writing, prior to implementation except as provided in Specification 6.8.3, with regard to whether or not each item considered under Specification 6.5.1.6a, through e, constitutes an unreviewed safety question.~~
- c- ~~Provide written notification within 24 hours to the Vice President Operations and the Safety Review Committee of disagreements between the PORC and the General Manager Plant Operations; however, the General Manager Plant Operations shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.~~

ADMINISTRATIVE CONTROLS

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- ~~a. Nuclear power plant operations;~~
- ~~b. Nuclear engineering;~~
- ~~c. chemistry and radiochemistry;~~
- ~~d. Metallurgy;~~
- ~~e. Instrumentation and control;~~
- ~~f. Radiological safety;~~
- ~~g. Mechanical and electrical engineering and~~
- ~~h. Quality assurance practices.~~

COMPOSITION

~~6.5.2.2 The SRC shall be composed of at least five members, including the Chairman. Members of the SRC may be from within the LP&L organization or from organizations external to LP&L.~~

~~The qualifications of members selected for the SRC shall be in accordance with Section 4.7 of ANSI/ANS 3.1-1978.~~

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ADMINISTRATIVE CONTROLS

MEETING FREQUENCY

~~6.5.2.5 The SRC shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.~~

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REVIEW

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- ~~a. The safety evaluations for (1) changes to procedures, equipment, or systems; and (2) tests or experiments completed under the provision of 10 CFR 50.59, to verify that such actions did not constitute an unreviewed safety question;~~
- ~~b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;~~
- ~~c. Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;~~
- ~~d. Proposed changes to Technical Specifications or this Operating License;~~
- ~~e. Violations of codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;~~
- ~~f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;~~
- ~~g. All REPORTABLE EVENTS;~~
- ~~h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems, or components that could affect nuclear safety; and~~
- ~~i. Reports and meeting minutes of the PORC~~

ADMINISTRATIVE CONTROLS

AUDITS

~~6.5.2.8 Audits of unit activities shall be performed under the cognizance of SRC. These audits shall encompass:~~

- ~~a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.~~
- ~~b. The performance, training, and qualifications of the entire unit staff at least once per 12 months.~~
- ~~c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety at least once per 6 months.~~
- ~~d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50, at least once per 24 months.~~
- ~~e. Any other area of unit operation considered appropriate by the SRC or the Vice President Operations.~~
- ~~f. The fire protection programmatic controls including the implementing procedures at least once per 24 months by qualified licensee QA personnel.~~
- ~~g. The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified off-site licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year.~~
- ~~h. The Primary Coolant Sources Outside Containment Program at least once per 24 months.~~
- ~~i. The In-Plant Radiation Monitoring Program at least once per 24 months.~~
- ~~j. The Secondary Water Chemistry Program at least once per 24 months.~~
- ~~k. The Post-Accident Sampling Program at least once per 24 months.~~
- ~~l. The Monitoring Program at least once per 24 months.~~
- ~~m. The radiological environmental monitoring program and the results thereof at least once per 12 months.~~
- ~~n. The OFF-SITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months.~~
- ~~o. The PROCESS CONTROL PROGRAM and implementing procedures for processing and packaging of radioactive wastes at least once per 24 months.~~

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- p- The performance of activities required by the Quality Assurance Program to meet the provisions of Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975, at least once per 12 months.

AUTHORITY

~~6.5.2.9 The SRC shall report to and advise the Vice President Operations on those areas of responsibility specified in Specifications 6.5.2.7 and 6.5.2.8.~~

RECORDS

~~6.5.2.10 Records of SRC activities shall be prepared, approved, and distributed as indicated below:~~

- ~~a- Minutes of each SRC meeting shall be prepared, approved, and forwarded to the Vice President Operations within 14 days following each meeting.~~
- ~~b- Reports of reviews encompassed by Specification 6.5.2.7 shall be prepared, approved, and forwarded to the Vice President Operations within 14 days following completion of the review.~~
- ~~c- Audit reports encompassed by Specification 6.5.2.8 shall be forwarded to the Vice President Operations and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.~~

TECHNICAL REVIEW AND CONTROL PROCESS

6.5.3 Procedures required by Specification 6.8.1 and other procedures which affect nuclear safety, as determined by the General Manager Plant Operations, and changes thereto, shall be reviewed as follows:

- a. Each procedure or change shall be independently reviewed by a qualified individual knowledgeable in the area affected other than the individual who prepared the procedure or procedure change. This review shall include a determination of whether or not additional cross disciplinary reviews are necessary. If deemed necessary, the reviews shall be performed by the review personnel of the appropriate discipline(s).
- b. Individuals performing these reviews shall meet the applicable qualifications of ANSI/ANS 3.1-1978, Section 4.0, excluding subsections 4.5.2 and 4.5.3, and be approved to perform these reviews in a given area by the General Manager Plant Operations.
- c. Those procedures and programs specified by Specification 6.5.1.6, and changes in intent thereto, shall be reviewed by PORC and approved by the General Manager Plant Operations prior to implementation.

Pages 6-9
through
page 6-12
not used

ADMINISTRATIVE CONTROLS

TECHNICAL REVIEW AND CONTROL PROCESS (Continued)

- d. Those procedures and programs specified by Specification 6.8.1, and changes in intent thereto, with the exception of those specified in Specification 6.5.1.6, shall be approved by the appropriate Group Head as specified in station administrative procedures.
- e. Review of the procedure or procedure change will include a determination of whether or not an unreviewed safety question is involved. This determination will be based on the review of a written safety evaluation prepared by a qualified individual, or documentation that a safety evaluation is not required. PORC review, SRC review, and NRC approval of items involving unreviewed safety questions shall be obtained prior to station approval for implementation.
- f. Written records of reviews performed in accordance with this specification shall be prepared and maintained in accordance with Specification 6.10.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the Plant Operations Review Committee (PORC) and the results of this review shall be submitted to the Safety Review Committee (SRC) and the Vice President Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Vice President Operations and the SRC shall be notified within 24 hours.
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PORC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence.
- c. The Safety Limit Violation Report shall be submitted to the Commission, the SRC and the Vice President Operations within 14 days of the violation.
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978 and Emergency Operating Procedures required to implement the requirements of NUREG 0737 and NUREG 0737, Supplement 1, as stated in Generic Letter 82-33.
- b. Refueling operations.
- c. Surveillance and test activities of safety-related equipment.
- d. Security Plan implementation Not used.
- e. Emergency Plan implementation Not used.
- f. Fire Protection Program implementation.
- g. Modification of Core Protection Calculator (CPC) Addressable Constants, including independent verification of modified constants.

NOTES:

(1) Modification to the CPC addressable constants based on information obtained through the Plant Computer - CPC data link shall not be made without prior approval of the PORC.

(2) Modifications to the CPC software (including algorithm changes and changes in fuel cycle specific data) shall be performed in accordance with the most recent version of CEN-39(A)P, "CPC Protection Algorithm Software Change Procedure," that has been determined to be applicable to the facility. Additions or deletions to CPC Addressable Constants or changes to Addressable Constant software limits values shall not be implemented without prior NRC approval.

- h. Administrative procedures implementing the overtime guidelines of Specification 6.2.2f., including provisions for documentation of deviations.
- i. PROCESS CONTROL PROGRAM implementation.

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PROCEDURES AND PROGRAMS (Continued)

- j. OFFSITE DOSE CALCULATION MANUAL implementation.
- k. Quality Assurance Program for effluent and environmental monitoring, using the guidance in Regulatory Guide 1.21, Revision 1, June 1974 and Regulatory Guide 4.1, Revision 1, April 1975.

6.8.2 Each procedure of Specification 6.8.1, and changes thereto, shall be reviewed and approved as required by ~~6-5~~ prior to implementation and reviewed periodically as set forth in administrative procedures.

6.8.3 Temporary changes to procedures of Specification 6.8.1 may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected;
- c. The change is documented, reviewed and approved as required by administrative procedures 6-5 within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

a. Primary Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include the containment spray safety injection, hydrogen analyzer, and the post-accident sampling system. The program shall include the following:

1. Preventive maintenance and periodic visual inspection requirements, and
2. Integrated leak test requirements for each system at refueling cycle intervals or less.

b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- i. Records of quality assurance activities required by the Quality Assurance Program Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PORC and the SRC.
- l. Records of the service lives of all hydraulic and mechanical snubbers required by Specification 3.7.8 including the date at which the service life commences and associated installation and maintenance records.
- m. Records of secondary water sampling and water quality.
- n. ~~DELETED~~ Records of audits performed under the requirements of Specification ~~6.5.2.8.~~
- o. Records of analyses required by the radiological environmental monitoring program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- p. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.

6.11 RADIATION PROTECTION PROGRAM

6.11.1 Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR Part 20, each high radiation area in which the intensity of radiation is greater than 100 mrem/h but less than 1000 mrem/h shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP)*. Any individual or group of individuals permitted to enter such areas shall be provided with or accompanied by one or more of the following:

- a. A radiation monitoring device which continuously indicates the radiation dose rate in the area.

*Health physics personnel or personnel escorted by health physics personnel shall be exempt from the RWP issuance requirement during the performance of their assigned radiation protection duties, provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.

TABLE 3.3-1 (Continued)

TABLE NOTATION

*With the protective system trip breakers in the closed position, the CEA drive system capable of CEA withdrawal, and fuel in the reactor vessel.

#The provisions of Specification 3.0.4 are not applicable.

**Not applicable above $10^{-4}\%$ RATED THERMAL POWER.

- (a) Trip may be manually bypassed above $10^{-4}\%$ of RATED THERMAL POWER; bypass shall be automatically removed when THERMAL POWER is less than or equal to $10^{-4}\%$ of RATED THERMAL POWER.
- (b) Trip may be manually bypassed below 400 psia; bypass shall be automatically removed whenever pressurizer pressure is greater than or equal to 500 psia.
- (c) Trip may be manually bypassed below $10^{-4}\%$ of RATED THERMAL POWER. Bypass shall be automatically removed when THERMAL POWER is greater than or equal to $10^{-4}\%$ of RATED THERMAL POWER. During testing pursuant to Special Test Exception 3.10.3, trip may be manually bypassed below 5% of RATED THERMAL POWER; bypass shall be automatically removed when THERMAL POWER is greater than or equal to 5% of RATED THERMAL POWER.
- (d) Trip may be bypassed during testing pursuant to Special Test Exception 3.10.3.
- (e) See Special Test Exception 3.10.2.
- (f) Each channel shall be comprised of two trip breakers, actual trip logic shall be one-out-of-two taken twice.
- (g) High steam generator level trip may be manually bypassed in Modes 1 and 2, at 20% power and below.

ACTION STATEMENTS

ACTION 1 - With the number of channels OPERABLE one less than required by the Minimum Channels OPERABLE requirement, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and/or open the protective system trip breakers.

ACTION 2 - With the number of channels OPERABLE one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may continue Provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. If the inoperable channel is bypassed, the desirability of maintaining this channel in the bypassed condition shall be documented by the Plant Operations Review Committee reviewed in accordance with plant administrative procedures Specification 6.5.1.6k. The channel shall be returned to OPERABLE status prior to STARTUP following the next COLD SHUTDOWN.

TABLE 3.3-3 (Continued)

TABLE NOTATION

- (a) Trip function may be bypassed in this MODE when pressurizer pressure is less than 400 psia; bypass shall be automatically removed when pressurizer pressure is greater than or equal to 500 psia.
- (b) An SIAS signal is first necessary to enable CSAS logic.
- * The provisions of Specification 3.0.4 are not applicable.

ACTION STATEMENTS

ACTION 12 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

ACTION 13 - With the number of channels OPERABLE one less than the Total Number of Channels, STARTUP and/or POWER OPERATION and/or operation in the other applicable MODE(S) may continue provided the inoperable channel is placed in the bypassed or tripped condition within 1 hour. If the inoperable channel is bypassed, the desirability of maintaining this channel in the bypassed condition shall be documented by the Plant Operations Review Committee reviewed in accordance with plant administrative procedures Specification 6.5.1.6k. The channel shall be returned to OPERABLE status no later than prior to entry into the applicable MODE(S) following the next COLD SHUTDOWN.

With a channel process measurement circuit that affects multiple functional units inoperable or in test, bypass or trip all associated functional units as listed below:

	Process Measurement Circuit	Functional Unit Bypassed/Tripped
1.	Containment Pressure - High	Containment Pressure - High (ESF) Containment Pressure - High (RPS)
2.	Steam Generator Pressure - Low	Steam Generator Pressure - Low Steam Generator P 1 and 2 (EFAS)
3.	Steam Generator Level	Steam Generator Level - Low Steam Generator Level - High Steam Generator P (EFAS)