

## LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK:                      (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	C	A	D	C	P	1	2	0	0	-	1	0	0	0	0	0	-	1	0	0	3	4	1	1	1	1	4	5						
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35						
LICENSEE CODE										LICENSE NUMBER										LICENSE TYPE										CAT 56				

01	0	1	0	5	0	0	0	2	7	5	7	0	5	1	0	8	3	8	0	5	2	3	8	3	9									
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31	32	33	34	35						
REPORT SOURCE										DOCKET NUMBER										EVENT DATE										REPORT DATE				

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 Prior to fuel load, ultrasonic (UT) thickness measurements were taken on

03 reactor coolant piping welds to establish baseline data for subsequent NDE

04 inspections. During these examinations, a possible under size condition

05 was discovered on weld WIB-RC 2-17. Minimum wall thickness by UT measure-

06 ment indicated that the weld was 0.065 inches below the required thickness

07 of 2.215 inches. Reportable per technical specification section 6.9.1.12.i

08 This even in no way effects the public health and safety.

09	C	B	11	A	12	C	13	P	I	P	E	X	X	14	E	15	Z	16				
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25				
SYSTEM CODE			CAUSE CODE			CAUSE SUBCODE			COMPONENT CODE					COMP SUBCODE		VALVE SUBCODE						
EVENT YEAR			SEQUENTIAL REPORT NO			OCCURRENCE CODE			REPORT TYPE		REVISION NO											
18 3			10 10 6			0 1 1			T		0											
21 22			23 24 25 26			27 28 29			30 31		32 33											
ACTION TAKEN			FUTURE ACTION			EFFECT ON PLANT			SHUTDOWN METHOD			HOURS			ATTACHMENT SUBMITTED		NPRD-4 FORM SUB		PRIME COMP SUPPLIER		COMPONENT MANUFACTURER	
Z 18			B 19			Z 20			Z 21			0 0 0 0			Y 23		N 24		N 25		S 218 0	
33 34			35 36			37 38 39			40 41 42			43 44 45			46 47 48			49 50 51				

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 This is an "Interim Report". The cause is still under investigation. If it

11 is determined that the weld is below minimum wall thickness, an approved

12 ASME section XI repair will be conducted or an analysis will be used to

13 determine if the present condition is acceptable. Supplemental information

14 is attached.

15	B	28	0	0	0	29	N/A	30	C	31	During non-required NDE	32		
7	8	9	10	11	12	13	14	15	16	17	18	19		
FACILITY STATUS			% POWER			OTHER STATUS			METHOD OF DISCOVERY			DISCOVERY DESCRIPTION		
10 11 12			13 14 15			16 17 18			19 20 21			22 23 24		
25 26 27			28 29 30			31 32 33			34 35 36			37 38 39		
ACTIVITY			CONTENT			AMOUNT OF ACTIVITY			LOCATION OF RELEASE					
10 11 12			13 14 15			16 17 18			19 20 21					
22 23 24			25 26 27			28 29 30			31 32 33					
34 35 36			37 38 39			40 41 42			43 44 45					
PERSONNEL EXPOSURES			TYPE			DESCRIPTION								
10 11 12			13 14 15			16 17 18								
19 20 21			22 23 24			25 26 27								
28 29 30			31 32 33			34 35 36								
37 38 39			40 41 42			43 44 45								
PERSONNEL INJURIES			TYPE			DESCRIPTION								
10 11 12			13 14 15			16 17 18								
19 20 21			22 23 24			25 26 27								
28 29 30			31 32 33			34 35 36								
37 38 39			40 41 42			43 44 45								
LOSS OF OR DAMAGE TO FACILITY			TYPE			DESCRIPTION								
10 11 12			13 14 15			16 17 18								
19 20 21			22 23 24			25 26 27								
28 29 30			31 32 33			34 35 36								
37 38 39			40 41 42			43 44 45								
PUBICITY			DESCRIPTION											
10 11 12			13 14 15			16 17 18								
19 20 21			22 23 24			25 26 27								
28 29 30			31 32 33			34 35 36								
37 38 39			40 41 42			43 44 45								

20 Y 44 PGandE News Release on May 13, 1983

NAME OF PREPARER William J. Kelly

PHONE 805/595-7351

RECEIVED  
MAY 25 11:02  
1383 MAY 25 11:02  
910-372-6587  
REGION VICE

# PACIFIC GAS AND ELECTRIC COMPANY

PG&E

77 BEALE STREET • SAN FRANCISCO, CALIFORNIA 94106 • (415) 781-4225

JAMES D. SHIFFER  
MANAGER

DEPARTMENT OF NUCLEAR PLANT OPERATIONS  
NUCLEAR POWER GENERATION

May 23, 1983

JDK  
Mr. John B. Martin, Regional Administrator  
U.S. Nuclear Regulatory Commission, Region V  
1450 Maria Lane, Suite 210  
Walnut Creek, CA 94596-5368

Re: Docket No. 50-275, OL-DPR-76  
Diablo Canyon Unit 1  
Licensee Event Report 83-006/01T-0  
Weld in Reactor Coolant Loop No. 2

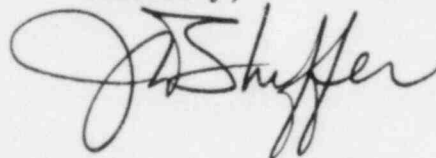
Dear Mr. Martin:

Pursuant to Section 6.9.1.12.i of the Technical Specifications, Appendix A to the Diablo Canyon Unit 1 Operating License, the enclosed Licensee Event Report is submitted concerning a weld on the Reactor Coolant Piping Loop No. 2 that is below the minimum wall thickness.

A follow-up report concerning this event will be submitted to your office at a later date.

This event has in no way affected public health and safety.

Sincerely,



JDS:vk

Enclosure

cc w/enc: Mr. George W. Knighton, Chief  
Licensing Branch No. 3  
Division of Licensing  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Director, Office of Management Information  
and Program Control  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Service List

IE-22  
83-111

ATTACHMENT TO LER 83-006/01T-0  
PACIFIC GAS AND ELECTRIC COMPANY  
DIABLO CANYON UNIT 1  
DOCKET NO. 50-275

SUPPLEMENTAL INFORMATION TO CAUSE DESCRIPTION.

There are four (4) areas on Weld No. WIB-RC-2-17 for which the NDE data indicates to be below the minimum wall thickness of 2.215 inches. Looking toward the reactor vessel and starting from Top Dead Center in the clockwise direction they are:

<u>AREA</u>	<u>LENGTH (inches)</u>	<u>THICKNESS (inches)</u>
1	2-5	2.190
2	7-18	2.150
3	38-49	2.170
4	82-85	2.190