NRC FORM 366
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | C A D C P 1 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | | (5)
LICENSEE CODE | LICENSE NUMBER | LICENSE TYPE | CAT

CON'T 0 1 REPORT SOURCE L (6) | 0 5 0 0 0 2 7 5 (7) | 0 5 0 9 8 3 (8) | 0 5 2 3 8 3 (9)
DOCKET NUMBER | EVENT DATE | REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 Prior to fuel load, while welding a reinforcement pad on a branch connec-
0 3 tion to the non-vital loop C component cooling water system,water began
0 4 leaking from the weld area. This event has in no way affected public
0 5 health and safety. Reportable per technical specification section
0 6 6.9.1.12.i.
0 7
0 8

0 9

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
W B (11)	A (12)	E (13)	P I P E X X (14)	A (15)	Z (16)

(17) LER/RO REPORT NUMBER

EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
8 3	—	0 0 7	/	0 1	T	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
Z (18)	B (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	N (24)	L (25)	X 9 9 9 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 This is an "Interim Report". Basic cause of the leak was a flaw in the
1 1 root pass of the shop weld joining the branch connection to the main
1 2 header. Removal of the fillet overweld while preparing the weld area for
1 3 addition of the reinforcement pad provided a leakage path for the water.
1 4 Repairs will be made using an approved ASME section XI repair program.

1 5 FACILITY STATUS: B (28) | % POWER: 0 0 0 (29) | OTHER STATUS (30): N/A | METHOD OF DISCOVERY: A (31) | DISCOVERY DESCRIPTION (32): During construction

1 6 ACTIVITY RELEASED: Z (33) | CONTENT OF RELEASE: Z (34) | AMOUNT OF ACTIVITY (35): N/A | LOCATION OF RELEASE (36): N/A

PERSONNEL EXPOSURES
1 7 NUMBER: 0 0 0 (37) | TYPE: Z (38) | DESCRIPTION (39): N/A

PERSONNEL INJURIES
1 8 NUMBER: 0 0 0 (40) | DESCRIPTION (41): N/A

LOSS OF OR DAMAGE TO FACILITY (43)
1 9 TYPE: Z (42) | DESCRIPTION: N/A

PUBLICITY
2 0 ISSUED: Y (44) | DESCRIPTION (45): PGandE News Release on May 13, 1983 | NRC USE ONLY

NAME OF PREPARER William J. Kelly PHONE 805/595-7351

ATTACHMENT TO LER 83-007/01T-0
PACIFIC GAS AND ELECTRIC COMPANY
DIABLO CANYON UNIT 1
DOCKET NO. 50-275

SUPPLEMENTAL INFORMATION

Ultrasonic examination and evaluation of ten similar welds without a reinforcement pad indicated the possibility that a full penetration shop weld was not made as required by Code. The affected portion of the component cooling water system was drained and the branch line connection removed for examination. A full penetration weld was present, with a small area (approximately 1/16 inch) where full penetration was not achieved at the weld root. Grinding on the weld surface in preparation for the installation of a reinforcing pad provided a leakage path for water. Repair to this weld will be made in accordance with an approved ASME Section XI repair program.

Presence of the full penetration weld in the CCW branch connection affects the credibility of the results of UT examination of the ten similar welds. Investigation into this matter will continue and results will be submitted in a revision to the LER.

PACIFIC GAS AND ELECTRIC COMPANY

PG&E
77 BEALE STREET • SAN FRANCISCO, CALIFORNIA 94106 • (415) 781-4211 • TWX 910-372-6587

JAMES D. SHIFFER
MANAGER
DEPARTMENT OF NUCLEAR PLANT OPERATIONS
NUCLEAR POWER GENERATION

May 23, 1983

Mr. John B. Martin, Regional Administrator
U.S. Nuclear Regulatory Commission, Region V
1450 Maria Lane, Suite 210
Walnut Creek, CA 94596-5368

Re: Docket No. 50-275, OL-DPR-76
Diablo Canyon Unit 1
Licensee Event Report 83-007/01T-0
Weld in Component Cooling Water System

Dear Mr. Martin:

Pursuant to Section 6.9.1.12.i of the Technical Specifications, Appendix A to the Diablo Canyon Unit 1 Operating License, the enclosed Licensee Event Report is submitted concerning a discrepancy in a shop weld in the Component Cooling Water System.

This event has in no way affected public health and safety.

Sincerely,

JDS:vk

Enclosure

cc w/enc: Mr. George W. Knighton, Chief
Licensing Branch No. 3
Division of Licensing
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Director, Office of Management Information
and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Service List

IE-22
83-110