

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01	N	C	M	G	S	2	0	0	-	0	0	0	0	0	0	0	3	4	1	1	1	1	4	5
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30	31
LICENSEE CODE													LICENSE NUMBER					LICENSE TYPE					CAT 58	

CONT

01	L	0	5	0	0	0	3	7	0	0	4	2	7	8	3	0	5	2	7	8	3	9	
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25	26	27	28	29	30
REPORT SOURCE		DOCKET NUMBER					EVENT DATE					REPORT DATE											

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While in Mode 3, investigation of control room indications determined that

03 main steam safety relief valve 2SV-20 lifted briefly while the main steam

04 pressure (app. 1105 psig) was below the acceptable lift setpoint range (1170

05 psig +/-1 percent). The valve was declared inoperable per T.S. 3.7.1.1 which

06 is reportable per T.S. 6.9.1.11(b). There were still 4 operable code safety

07 valves on steam generator 2A, and 2SV-20 was still capable of relieving steam

08 pressure. Health and safety of the public were unaffected.

09	C	C	11	E	12	B	13	V	A	L	V	E	X	14	J	15	B	16			
7	8	9	10	11	12	13	14	15	16	17	18	19	20	21	22	23	24	25			
SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE					COMP SUBCODE		VALVE SUBCODE								
EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE		REPORT TYPE		REVISION NO.													
17	8	3	1	2	3	1	1	0	0												
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD		HOURS		ATTACHMENT SUBMITTED		NPRD-4 FORM SUB		PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER					
E	18	Z	19	Z	20	Z	21	0	0	0	0	N	23	N	24	L	25	C	7	1	0
33	34	35	36	37	38	39	40	41	42	43	44	45	46	47	48	49	50	51	52	53	54

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Apparently the valve's (Crosby, nozzle type C safety valve, style HA-65W) set-

11 point drifted from its original setting of 1169 psig. The adjusting nut is

12 held stationary with a cotter pin, however minor drift (possibly due to vibra-

13 tion) is not uncommon. The valve's setpoint was adjusted to its correct value

14 Steam pressure was kept well below the lift setpoint until the setpoint was

15 corrected.

15	B	28	0	0	0	29	N/A	30	A	31	Control room indications	32
7	8	9	10	11	12	13	14	15	16	17	18	19
FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION				
ACTIVITY CONTENT RELEASED OF RELEASE		AMOUNT OF ACTIVITY		LOCATION OF RELEASE								
16	Z	33	Z	34	N/A	N/A						
7	8	9	10	11	12	13						
PERSONNEL EXPOSURES NUMBER		TYPE		DESCRIPTION								
17	0	0	0	37	Z	38	N/A					
7	8	9	10	11	12	13	14					
PERSONNEL INJURIES NUMBER		DESCRIPTION										
18	0	0	0	40	N/A							
7	8	9	10	11	12	13	14					
LOSS OF OR DAMAGE TO FACILITY TYPE		DESCRIPTION										
19	Z	42	N/A									
7	8	9	10	11	12	13	14					
PUBLICITY ISSUED DESCRIPTION		N/A										
20	N	44	N/A									
7	8	9	10	11	12	13	14					

NAME OF PREPARER Phillip B. NardociPHONE: (704) 373-7432

DUKE POWER COMPANY

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VICE PRESIDENT
NUCLEAR PRODUCTION

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May 27, 1983

Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street NW, Suite 2900
Atlanta, Georgia 30303

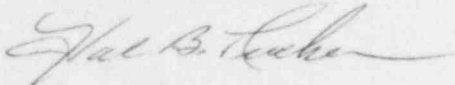
Re: McGuire Nuclear Station Unit 2
Docket No. 50-370

83 JUN 6 9:24
USNRC REGION II
ATLANTA, GEORGIA

Dear Mr. O'Reilly:

Please find attached Reportable Occurrence Report RO-370/83-12. This report concerns T.S. 3.7.1.1, "All main steam line code safety valves associated with each steam generator shall be operable with lift settings as specified in Table 3.7-3." This incident was considered to be of no significance with respect to the health and safety of the public.

Very truly yours,



Hal B. Tucker

PBN:jfw
Attachment

cc: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. W. T. Orders
NRC Resident Inspector
McGuire Nuclear Station

Records Center
Institute of Nuclear Power Operations
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