

**North  
Atlantic**

North Atlantic Energy Service Corporation  
P.O. Box 300  
Seabrook, NH 03874  
(603) 474-9521, Fax (603) 474-2987

The Northeast Utilities System

Ted C. Feigenbaum  
Senior Vice President &  
Chief Nuclear Officer

NYN- 94087

July 29, 1994

United States Nuclear Regulatory Commission  
Washington, D.C. 20555

Attention: Document Control Desk

- References:
- (a) Facility Operating License No. NPF-86, Docket No. 50-443
  - (b) North Atlantic Letter NYN-94073 dated June 29, 1994, "Generic Letter 92-01, Rev 1, Reactor Vessel Structural Integrity (TAC No. M83512)", T.C. Feigenbaum to USNRC
  - (c) USNRC Letter dated May 27, 1994, "Generic Letter 92-01, Rev 1, Reactor Vessel Structural Integrity (TAC No. M83512)," A.W. De Agazio to T.C. Feigenbaum

Subject: Generic Letter 92-01, Rev 1, Reactor Vessel Structural Integrity (TAC No. M83512)

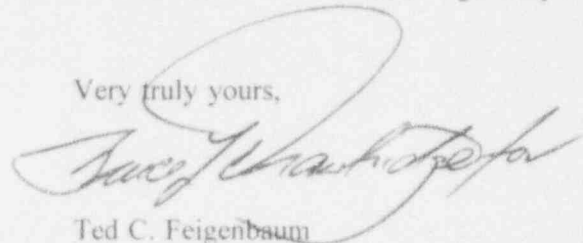
Gentlemen:

As previously reported in Reference (b), North Atlantic Energy Service Corporation (North Atlantic) determined that the values for copper and nickel content for the Seabrook Station reactor vessel as stated in the Summary File for Pressurized Thermal Shock provided in Reference (c) are not correct in all cases. The values listed in Reference (c) are those values supplied by the reactor vessel manufacturer, Combustion Engineering, and were contained in the previous submittals for Seabrook Station. However, the values contained in the initial material certifications supplied by the reactor vessel plate manufacturer Lukens Steel Company are slightly different in some cases. North Atlantic committed to providing a revised Summary File for Pressurized Thermal Shock using the mean of the copper and nickel values, with the revised Chemistry Factors, by July 29, 1994. The revised Summary File for Pressurized Thermal Shock is enclosed.

North Atlantic is reviewing other license documents to determine if they are affected by this change and will make appropriate changes, if required.

Should you have any questions regarding this matter, please contact Mr. James M. Peschel, Regulatory Compliance Manager at (603) 474-9521, extension 3772.

Very truly yours,



Ted C. Feigenbaum

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Enclosure

cc: Mr. Thomas T. Martin  
Regional Administrator  
U.S. Nuclear Regulatory Commission  
Region I  
475 Allendale Road  
King of Prussia, PA 19406

Mr. Albert W. De Agazio, Sr. Project Manager  
Project Directorate I-4  
Division of Reactor Projects  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Mr. Antone C. Cerne  
NRC Senior Resident Inspector  
P.O. Box 1149  
Seabrook, NH 03874

North Atlantic  
July 29, 1994

ENCLOSURE TO NYN-94087

## Summary File for Pressurized Thermal Shock

Plant Name	Beltline Ident.	Heat No. Ident.	ID Neut. Fluence at EOL/EPY	IRT <sub>max</sub>	Method of Determin. IRT <sub>max</sub>	Chemistry Factor	Method of Determin. CF	%Cu	%Ni
Seabrook  EOL: 10/17/ 2006- 2026	Lower Shell Plate	R1808-1	3.1E19	40°F	Plant Specific	<del>33</del> 37	Table	<del>0.05</del> .06	0.58
	Lower Shell Plate	R1808-2	3.1E19	10°F	Plant Specific	<del>33</del> 37	Table	<del>0.05</del> .06	<del>0.57</del> .58
	Lower Shell Plate	R1808-3	3.1E19	40°F	Plant Specific	<del>37</del> 44	Table	<del>0.06</del> .07	<del>0.57</del> .59
	Int. Shell Plate	R1806-1	3.1E19	40°F	Plant Specific	<del>26</del> 28.5	Table	<del>0.04</del> .045	<del>0.64</del> .61
	Int. Shell Plate	R1806-2	3.1E19	0°F	Plant Specific	<del>34</del> 37	Table	<del>0.05</del> .06	<del>0.65</del> .64
	Int. Shell Plate	R1806-3	3.1E19	10°F	Plant Specific	<del>44</del> 47.5	Table	<del>0.07</del> .075	<del>0.65</del> .63
	Int. Shell Axial Welds 101-124A/C	4P6052	3.1E19	-60°F	Plant Specific	34.3	Table	0.07	0.02
	Lower Shell Axial Welds 101-142A/C	4P6052	3.1E19	-60°F	Plant Specific	34.3	Table	0.07	0.02
	Int. to Lower Shell Circ. Weld 101-171	4P6052	3.1E19	-60°F	Plant Specific	34 "	Table	0.07	0.02

References for Seabrook

IRT<sub>max</sub> and chemical composition data are from the August 17, 1992, letter from T. C. Feigenbaum (PSNH) to USMRC Document Control Desk, subject: Reactor Vessels Capsule Report.

Fluence from August 17, 1992 PTS submittal.