

LICENSEE EVENT REPORT

CONTROL BLOCK: ①

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

① ② I L D R S 2 ③ 0 0 - 0 0 0 0 0 - 0 0 ④ 4 1 1 1 1 ⑤ ⑥ ⑦ ⑧ ⑨

LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 37 CAT 58

CON'T

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REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

① ② While increasing power, drywell floor drain leakage was calculated to be in
③ ④ excess of 3 gpm (T.S. 4.6.D.2.a). Upon exceeding 4 gpm, a drywell inspection
⑤ ⑥ was conducted revealing bonnet leakage on feedwater check valve 2-220-58B.
⑦ ⑧ Normal unit shutdown was initiated to make repairs. The event was of minimal
⑨ ⑩ safety significance since all leakage was contained in the drywell floor drain
⑪ ⑫ sump. No effect on public health or safety. This was the first reportable
⑬ ⑭ occurrence of this type.

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SYSTEM CODE 9 10 CAUSE CODE 11 CAUSE SUBCODE 12 COMPONENT CODE 13 COMP. SUBCODE 14 VALVE SUBCODE 15

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LER RO REPORT NUMBER 21 22 EVENT YEAR 23 24 SEQUENTIAL REPORT NO. 25 26 OCCURRENCE CODE 27 28 REPORT TYPE 29 30 REVISION NO. 31 32

ACTION TAKEN 33 FUTURE ACTION 34 EFFECT ON PLANT 35 SHUTDOWN METHOD 36 HOURS 37 38 ATTACHMENT SUBMITTED 39 40 NPRO-4 FORM SUP. 41 PRIME COMP. SUPPLIER 42 COMPONENT MANUFACTURER 43 44 45 46 47 48 49 50

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ⑳

① ② The leakage was caused by a failed bonnet seal ring. The seal ring was sub-
③ ④ sequently replaced. Drywell floor drain leakage will continue to be
⑤ ⑥ monitored to verify system integrity.

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FACILITY STATUS 28 29 % POWER 30 31 OTHER STATUS 32 33 METHOD OF DISCOVERY 34 35 DISCOVERY DESCRIPTION 36 37

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ACTIVITY CONTENT 38 39 RELEASED OF RELEASE 40 41 AMOUNT OF ACTIVITY 42 43 LOCATION OF RELEASE 44 45

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PERSONNEL EXPOSURES 46 47 NUMBER 48 49 TYPE 50 51 DESCRIPTION 52 53

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PERSONNEL INJURIES 54 55 NUMBER 56 57 DESCRIPTION 58 59

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LOSS OF OR DAMAGE TO FACILITY 60 61 TYPE 62 63 DESCRIPTION 64 65

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PUBLICITY 66 67 ISSUED 68 69 DESCRIPTION 70 71

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ISSUED 72 73 DESCRIPTION 74 75

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PDR ADOCK 05000237
S PDR

NRC USE ONLY

NAME OF PREPARER R. ChristelPHONE: (815) 942-2920 x421



Commonwealth Edison

DEVIATION REPORT

DVR NO. 12 - 2 - 83 - 70
STA UNIT YEAR NO.

PART 1 TITLE OF DEVIATION

OCCURRED 5/2/83 1948
DATE TIME

Unidentified Drywell Leakage In Excess of 3 gpm

SYSTEM AFFECTED 1600

PLANT STATUS AT TIME OF EVENT

Containment

MODE Run

PWR(MWT) 1809

LOAD(MWE) 585

TESTING

☐ YES ☒ NO

DESCRIPTION OF EVENT At 1948 hours the unidentified leakage rate to the primary

containment was calculated to be 3.5 gpm. At 2235 hours calculations

revealed 4.43 gpm.

10 CFR50.72 NRC RED PHONE

☒

At 2014 hrs

NOTIFICATION MADE

YES

NO 5/2/83

EQUIPMENT FAILURE 27580

☒ YES☐ NO

WORK REQUEST NO.

RESPONSIBLE SUPERVISOR

Michael Wright

DATE 5/3/83

PART 2 OPERATING ENGINEER'S COMMENTS

Per Tech Spec 4.6.D.2, the containment was deinerted and a leak inspection

performed which revealed a bonnet leak on the feedwater check valve in

2B feedwater line. The NRC inspector was notified at about 2100, and

again at 2300 hours when the decision to shutdown was reached.

☐ EVENT OF PUBLIC INTEREST☐ TECH. SPEC. VIOLATION☐ NON REPORTABLE OCCURRENCE☐ 14 DAY REPORTABLE/T.S.☒ 30 DAY REPORTABLE/T.S. 6.6.B.2.d☐ ANNUAL/SPECL REPORT REQ'D☐ 24-HOUR NRC NOTIFICATION REQ'D (Courtesy)

TELEPH Tom Tongue

5/3/83

0900

REGION III

DATE

TIME

TELEGM/TELECOPY N/A

REGION III

DATE

TIME

☐ CECO CORPORATE NOTIFICATION MADE

IF ABOVE NOTIFICATION IS PER 10CFR21

☐ 5-DAY WRITTEN REPORT REQ'D PER 10CFR21

Telecopy Dennis P. Galle

5/3/83

1045

Teleph Les Bird

5/2/83

2300

CECO CORPORATE OFFICER

DATE

TIME

PRELIMINARY REPORT
COMPLETED AND REVIEWED

Michael Wright

5/3/83

OPERATING ENGINEER

DATE

INVESTIGATED REPORT & RESOLUTION
ACCEPTED BY STATION REVIEW

5/31/83

5-31-83

RESOLUTION APPROVED AND
AUTHORIZED FOR DISTRIBUTION

STATION SUPERINTENDENT

DATE



Commonwealth Edison

Dresden Nuclear Power Station

R.H. #1

Morris, Illinois 60450

Telephone 815/942-2920

May 31, 1983

DJS Ltr #83-524

James G. Keppler, Regional Administrator
Region III
U.S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

Reportable Occurrence Report #83-36/03L-0, Docket #050-237 is being submitted to your office in accordance with Dresden Nuclear Power Station Technical Specification 6.6.B.2.(d), abnormal degradation of systems other than those specified in item B.1.e. above designed to contain radioactive material resulting from the fission process.

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: Director of Inspection & Enforcement
Director of Management Information & Program Control
U.S. NRC, Document Management Branch
File/NRC

JUN 1 1983

IE 22