

DUKE POWER COMPANY

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NUCLEAR PRODUCTION

TELEPHONE
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May 27, 1983

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Attention: Ms. E. G. Adensam, Chief
Licensing Branch No. 4

Re: Catawba Nuclear Station
Docket Nos. 50-413 and 50-414

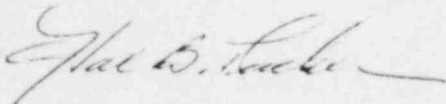
Dear Mr. Denton:

The purpose of this letter is to provide follow-up information in response to two items discussed in the Catawba Safety Evaluation Report.

Open Item 7 - Pressurizer safety valve sizing - As noted in Section 5.2.2.1 of the SER, the Staff has requested additional information on the details of this calculation. Information on pressurizer safety valve sizing has been provided in response to Questions 440.11, 124 and 130. This item was also discussed in detail at the ACRS Meeting on Catawba (275th General Meeting, March 11, 1983, transcript pages 206-220). In addition, please find attached the Overpressure Protection Report for Catawba which is required by Section III of the ASME Code. This report shows that the overpressure protection provided is sufficient to maintain peak Reactor Coolant System pressure below the code limit of 110% of system design pressure.

Confirmatory Item 24 - LOCA sensitivity analysis - As requested by Question 440.147 and as discussed in SER Section 6.3.5.1, additional DECLG break cases have been analyzed. The results confirm that the previously identified limiting break remains the limiting case. The response to Question 440.147 is attached.

Very truly yours,



Hal B. Tucker

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Attachment

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, NW, Suite 2900
Atlanta, Georgia 30303

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cc: Mr. P. K. Van Doorn
NRC Resident Inspector
Catawba Nuclear Station

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